

23

LAMS-2415

GIC-14 REPORT COLLECTION
REPRODUCTION
COPY

LOS ALAMOS SCIENTIFIC LABORATORY
OF THE UNIVERSITY OF CALIFORNIA ○ LOS ALAMOS NEW MEXICO

CRITICAL DATA FOR NUCLEAR SAFETY GUIDANCE

LOS ALAMOS NATIONAL LABORATORY



3 9338 00371 7492

LEGAL NOTICE

This report was prepared as an account of Government sponsored work. Neither the United States, nor the Commission, nor any person acting on behalf of the Commission:

A. Makes any warranty or representation, expressed or implied, with respect to the accuracy, completeness, or usefulness of the information contained in this report, or that the use of any information, apparatus, method, or process disclosed in this report may not infringe privately owned rights; or

B. Assumes any liabilities with respect to the use of, or for damages resulting from the use of any information, apparatus, method, or process disclosed in this report.

As used in the above, "person acting on behalf of the Commission" includes any employee or contractor of the Commission, or employee of such contractor, to the extent that such employee or contractor of the Commission, or employee of such contractor prepares, disseminates, or provides access to, any information pursuant to his employment or contract with the Commission, or his employment with such contractor.

Printed in USA. Price \$2.00. Available from the
Office of Technical Services
U. S. Department of Commerce
Washington 25, D. C.

LAMS-2415
CRITICALITY STUDIES
TID-4500, 15th Ed.

LOS ALAMOS SCIENTIFIC LABORATORY
OF THE UNIVERSITY OF CALIFORNIA LOS ALAMOS NEW MEXICO

REPORT COMPILED: February 1960

REPORT DISTRIBUTED: May 16, 1960

CRITICAL DATA FOR NUCLEAR SAFETY GUIDANCE

Compiled by

H. C. Paxton



Contract W-7405-ENG. 36 with the U. S. Atomic Energy Commission

All LAMS reports are informal documents, usually prepared for a special purpose and primarily prepared for use within the Laboratory rather than for general distribution. This report has not been edited, reviewed, or verified for accuracy. All LAMS reports express the views of the authors as of the time they were written and do not necessarily reflect the opinions of the Los Alamos Scientific Laboratory or the final opinion of the authors on the subject.



PREFACE

This collection of critical data is intended for the convenience of those who wish to evaluate nuclear safety problems. It is made available in the present form pending consideration as a supplement to TID 7016, THE NUCLEAR SAFETY GUIDE. Major sources of information outside of the Los Alamos Scientific Laboratory are:

The Argonne National Laboratory

The Dow Chemical Company, Rocky Flats Plant

General Electric Company, Hanford Atomic Products Operation

Lawrence Radiation Laboratory, Livermore

Union Carbide Corporation, Oak Ridge National Laboratory

U K Atomic Energy Research Establishment, Harwell

U K Atomic Weapons Research Establishment, Aldermaston.

ILLUSTRATIONS

	<u>Page</u>
Fig. 1. Critical masses of homogeneous water-moderated Oy(93.2) spheres	8
Fig. 2. Critical volumes of homogeneous water-moderated Oy(93.2) spheres	9
Fig. 3. Estimated critical diameters of infinite cylinders of homogeneous water-moderated Oy(93.2).	10
Fig. 4. Estimated critical thicknesses of infinite slabs of homogeneous water-moderated Oy(93.2)	11
Fig. 5. Critical masses of homogeneous water-moderated plutonium spheres.	12
Fig. 6. Critical volumes of homogeneous water-moderated plutonium spheres.	13
Fig. 7. Estimated critical diameters of infinite cylinders of homogeneous water-moderated Pu ²³⁹ (no nitrate)	14
Fig. 8. Estimated critical thicknesses of infinite slabs of homogeneous water-moderated Pu ²³⁹ (no nitrate)	15
Fig. 9. Critical masses of homogeneous water-moderated U ²³³ spheres	16
Fig. 10. Critical volumes of homogeneous water-moderated U ²³³ spheres	17
Fig. 11. Estimated critical diameters of infinite cylinders of homogeneous water-moderated U ²³³	18
Fig. 12. Estimated critical thicknesses of infinite slabs of homogeneous water-moderated U ²³³	19
Fig. 13. Minimum critical mass of flooded Oy(93.5) metal lattices as a function of oralloy unit size.	20
Fig. 14a. Minimum critical masses of water-moderated oralloy at reduced U ²³⁵ content.	21
Fig. 14b. Minimum critical volumes of water-moderated oralloy at reduced U ²³⁵ content.	22

ILLUSTRATIONS (Continued)

	<u>Page</u>
Fig. 15. Critical mass vs. U^{235} concentration of oralloy metal	23
Fig. 16. Critical mass vs. critical, water-reflected spheres of Pu solution containing excess nitrate	25
Fig. 17. Critical heights of 20"-diam. cylinders containing pyrex-poisoned solutions of $Oy(87.4)O_2(NO_3)_2$	27
Fig. 18. Critical masses of bare homogeneous spheres of $Oy(93)$ and moderators other than light water	31
Fig. 19. Spherical critical masses of Pu diluted with other metals.	32
Fig. 20. Critical masses of $Oy(93.5)$ metal in H_2O , D_2O , C, Al, and Fe reflectors	33
Fig. 21. Critical masses of $Oy(93.5)$ metal in U, W, Ni, Be, and BeO reflectors.	34
Fig. 22. Critical masses of U^{233} , δ - Pu^{239} and α - Pu^{239} in terms of $Oy(93.5)$ metal at same reflector composition and thickness	36
Fig. 23. Critical height of U^{235} -solution cylinder as function of thickness of water or furfural reflector on lateral surface	38
Fig. 24. Critical height of a 6"-thick slab of U^{235} solution vs. thickness of Al on each face of the slab.	39
Fig. 25. Critical volumes of cylinders of $Oy(93)O_2F_2$ solution relative to spherical values.	40
Fig. 26. Critical masses of cylinders of $Oy(93.5)$ metal relative to spherical values	41
Fig. 27. Effective extrapolation lengths for cylinders of $Oy(93.2)O_2F_2$ solutions	42
Fig. 28. Effective extrapolation lengths for cylinders of $Oy(93.5)$ and δ -phase Pu metal.	43

ILLUSTRATIONS (Continued)

	<u>Page</u>
Fig. 29. Critical heights of annuli containing U^{235} solution vs. thicknesses of the annuli, H/U^{235} atomic ratio = 50.4	44
Fig. 30. Critical heights of annuli containing U^{235} solution vs. thicknesses of the annuli, H/U^{235} atomic ratio = 309	45
Fig. 31. Critical thicknesses of 10" x 16" Oy-metal slabs in U^{235} solutions	49
Fig. 32. Estimated critical thicknesses of Oy-metal slabs in infinite U^{235} solutions.	50
Fig. 33. Critical thicknesses of 5" x 8" Oy-metal slabs in 9.45"-diam. U^{235} solutions	51
Fig. 34. Cubic arrays of Oy-metal units, critical lattice capacity vs. fraction of space occupied by units.	53
Fig. 35. Approximately cubic arrays of U^{235} solutions, critical lattice capacity vs. fraction of space occupied by units.	54
Fig. 36. Density exponents for cubic arrays as functions of unit size	56
Fig. 37. Linear and planar arrays of Oy-metal units, interaction vs. number of units.	57
Fig. 38. Interaction within in-plane array of U^{235} -solution cylinders.	58
Fig. 39. Interaction between flooded pairs of units vs. separation.	59
Fig. 40. Critical mass of U^{235} -solution cylinder as function of distance from a concrete wall.	60
Fig. 41. Critical mass of U^{235} -solution cylinder as function of thickness of carbon on its base.	62
Fig. 42. Critical masses of U^{235} -solution cylinder for various combinations of carbon and firebrick on its base	63

ILLUSTRATIONS (Continued)

	<u>Page</u>
Fig. 43. Reactivity of plane array of Oy-metal units vs. thickness of nearby concrete wall	64
Fig. 44. Interaction between two plane arrays of Oy-metal units vs. thickness of intervening concrete	65

INDIVIDUAL UNITS

Homogeneous, water-moderated systems

Figures 1 and 2 represent critical masses and critical volumes of homogeneous, water-moderated spheres of Oy(93.2), both bare (except for thin-wall container) and water-reflected.^(1-6,31) Estimates of corresponding diameters of infinite critical cylinders appear in Figure 3, and thicknesses of infinite critical slabs in Figure 4.^(4,6,11,12,32) Effective extrapolation lengths of Figures 27 and 28 are used for the shape conversions that are involved. Similar data for water-moderated Pu²³⁹ appear in Figures 5-8,^(4,6-8,32,33) and for U²³³ in Figures 9-12.^(4,6,8-10,34) The idealized metal-water mixtures of Figures 1-12 (> 2 kg/liter) are denser, hence more limiting, than usually encountered.

Inhomogeneous water-moderated Oy

Figure 13 shows how the minimum critical mass of a water-moderated, water-reflected lattice of Oy(93.5) pieces (optimum spacing) depends upon size of piece.^(26,35) Though measurements were on 1" cubes, 1/2" cubes, and 1/8" diameter rods, data appear in terms of approximate diameters of equivalent spheres. Surface-to-surface spacings that correspond to minima in critical mass vary from 0.7" for the 1" cubes to 0.6" for the 1/8" rods.

Oy at reduced U²³⁵ content

Minimum critical masses of homogeneous, water-moderated, water-reflected Oy are given as functions of U²³⁵ content of the Oy in Figure 14a.^(24,36-38) Also shown are minimum critical masses of water-moderated lattices in the enrichment range through which these critical masses are less than those for homogeneous systems.^(29,39) Similarly, Figure 14b displays minimum critical volumes. Critical masses of unmoderated Oy(93.5) metal vs. U²³⁵ concentration appear in Figure 15.⁽⁶⁾

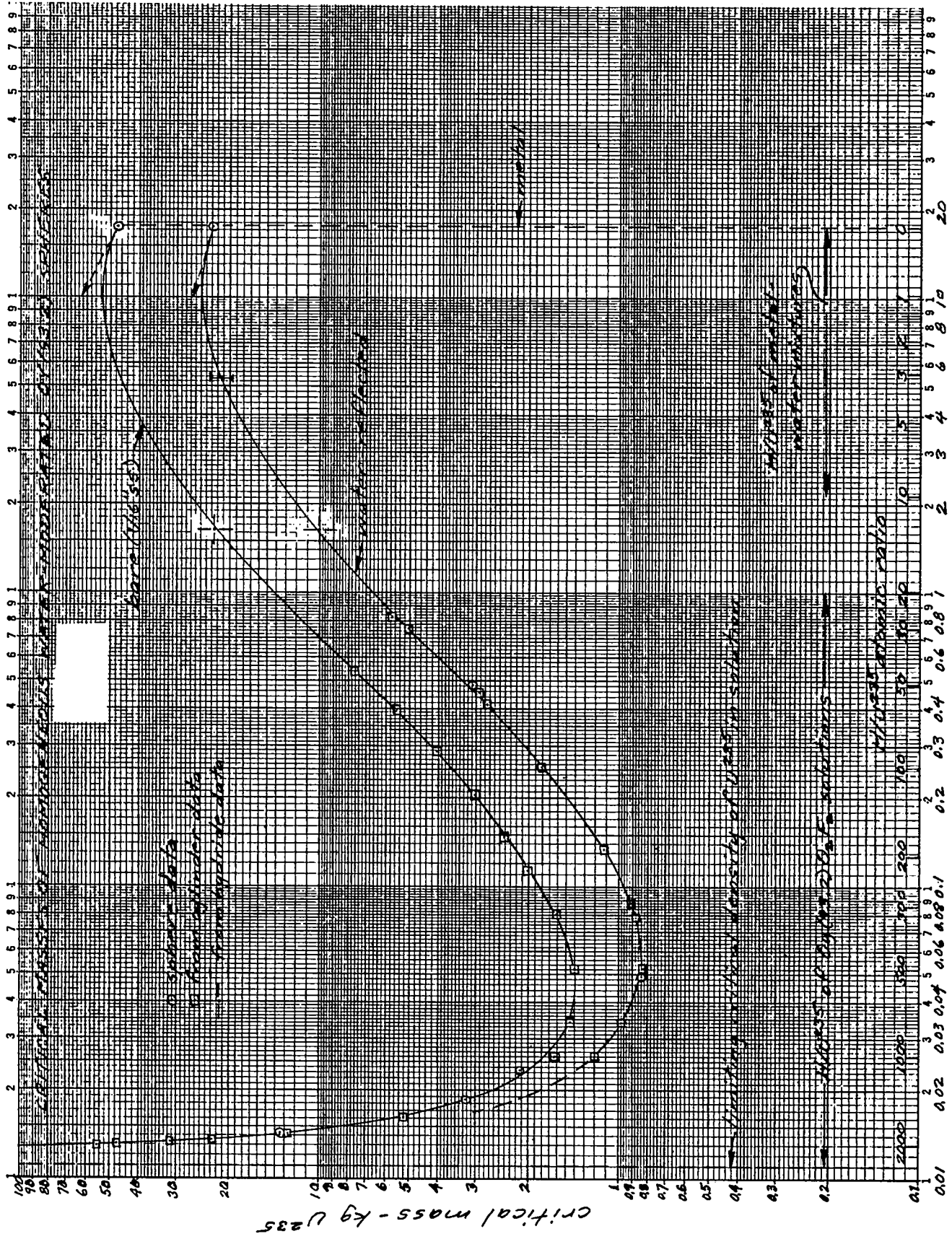


Fig. 1. - 8 -

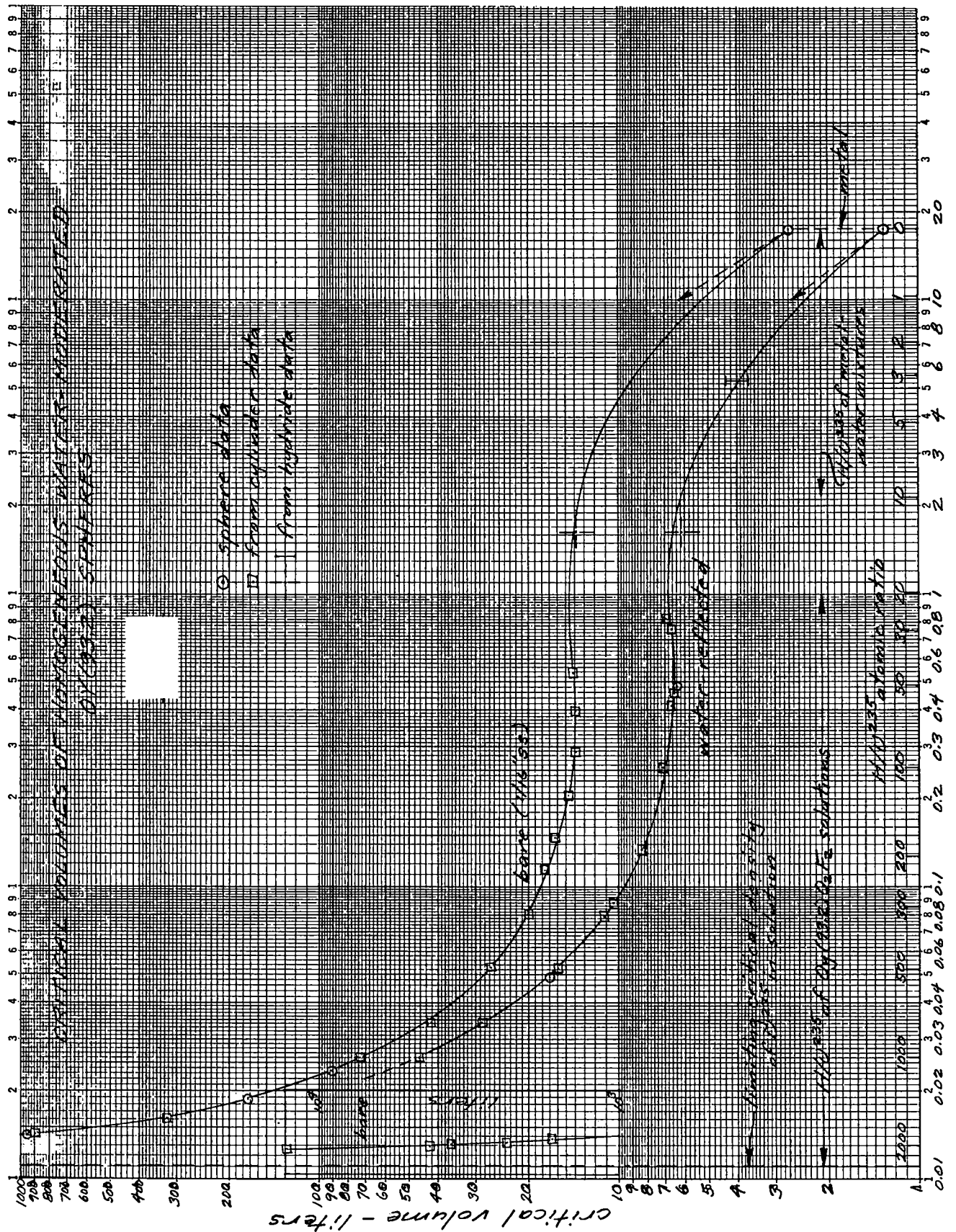


Fig. 2. - 9 -

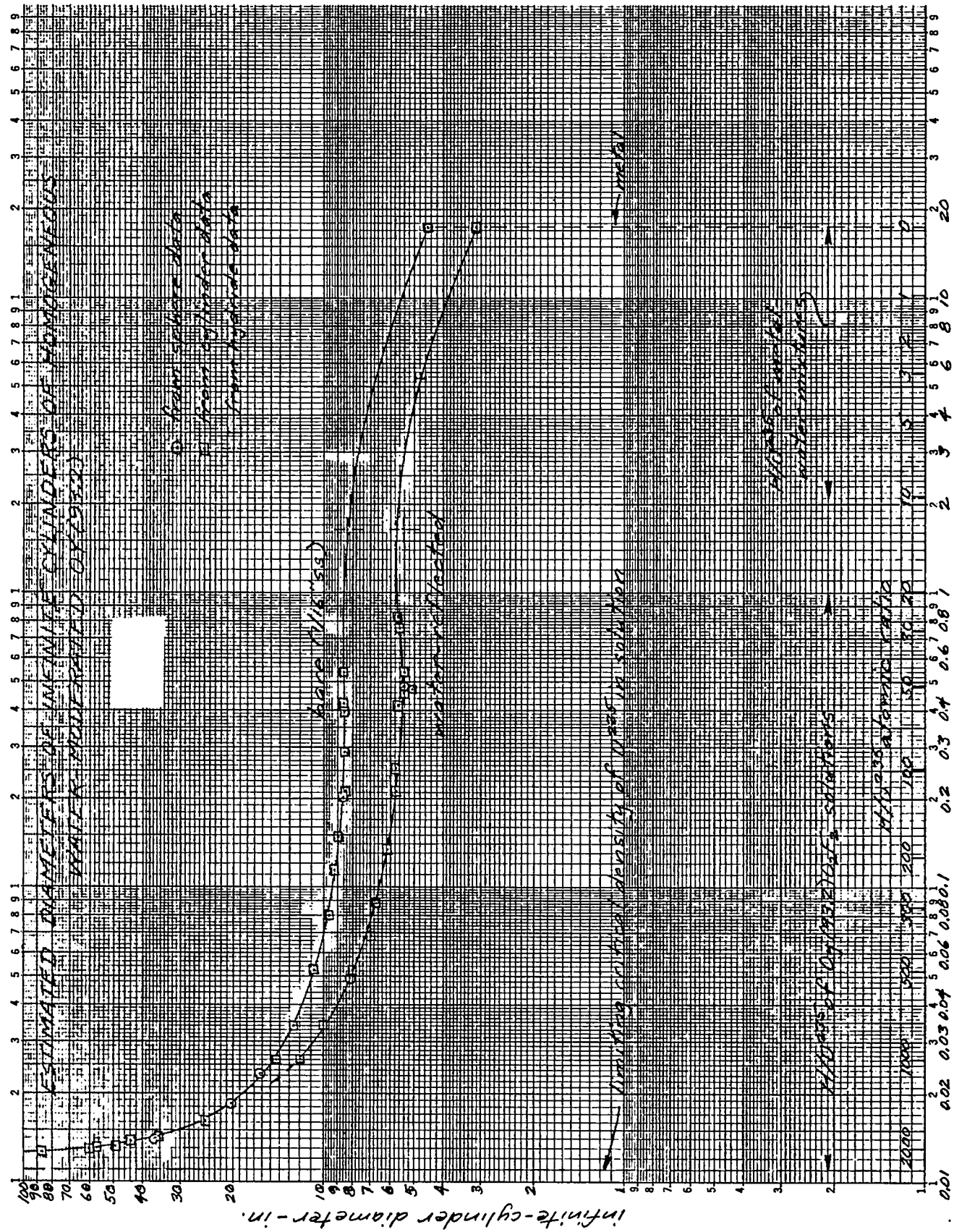


Fig. 3. - 10 -

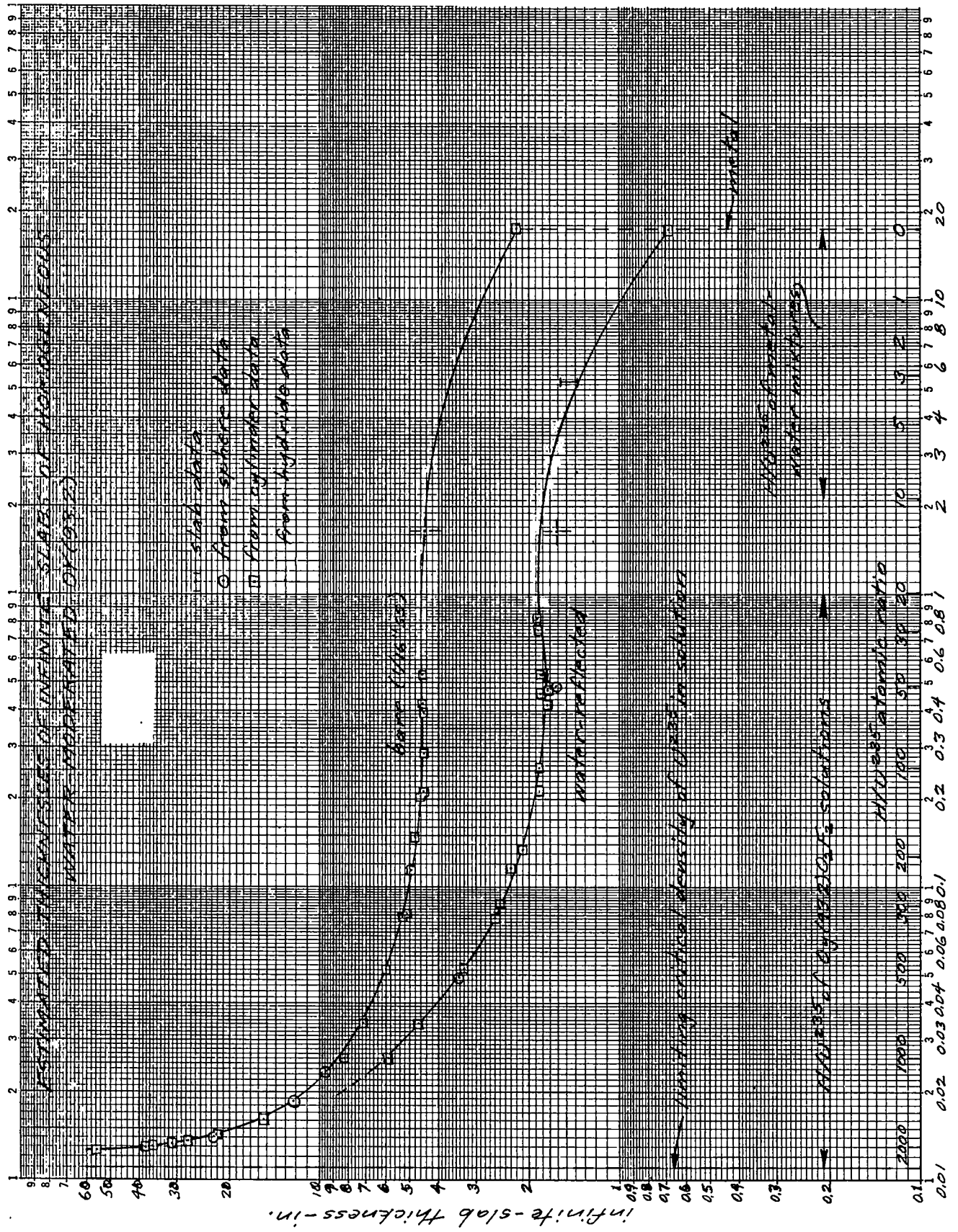


Fig. 4. - 11 -

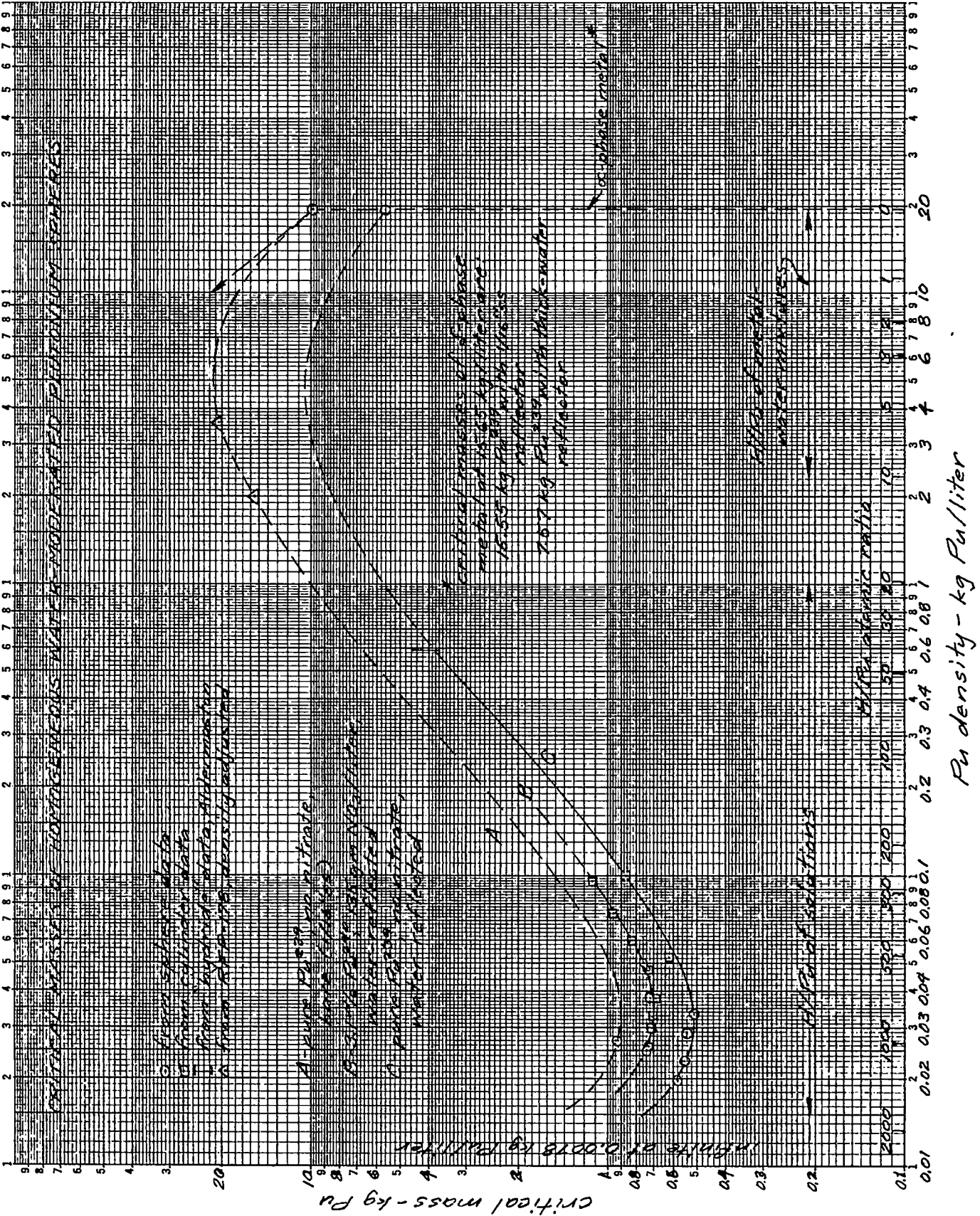


Fig. 5. - 12 -

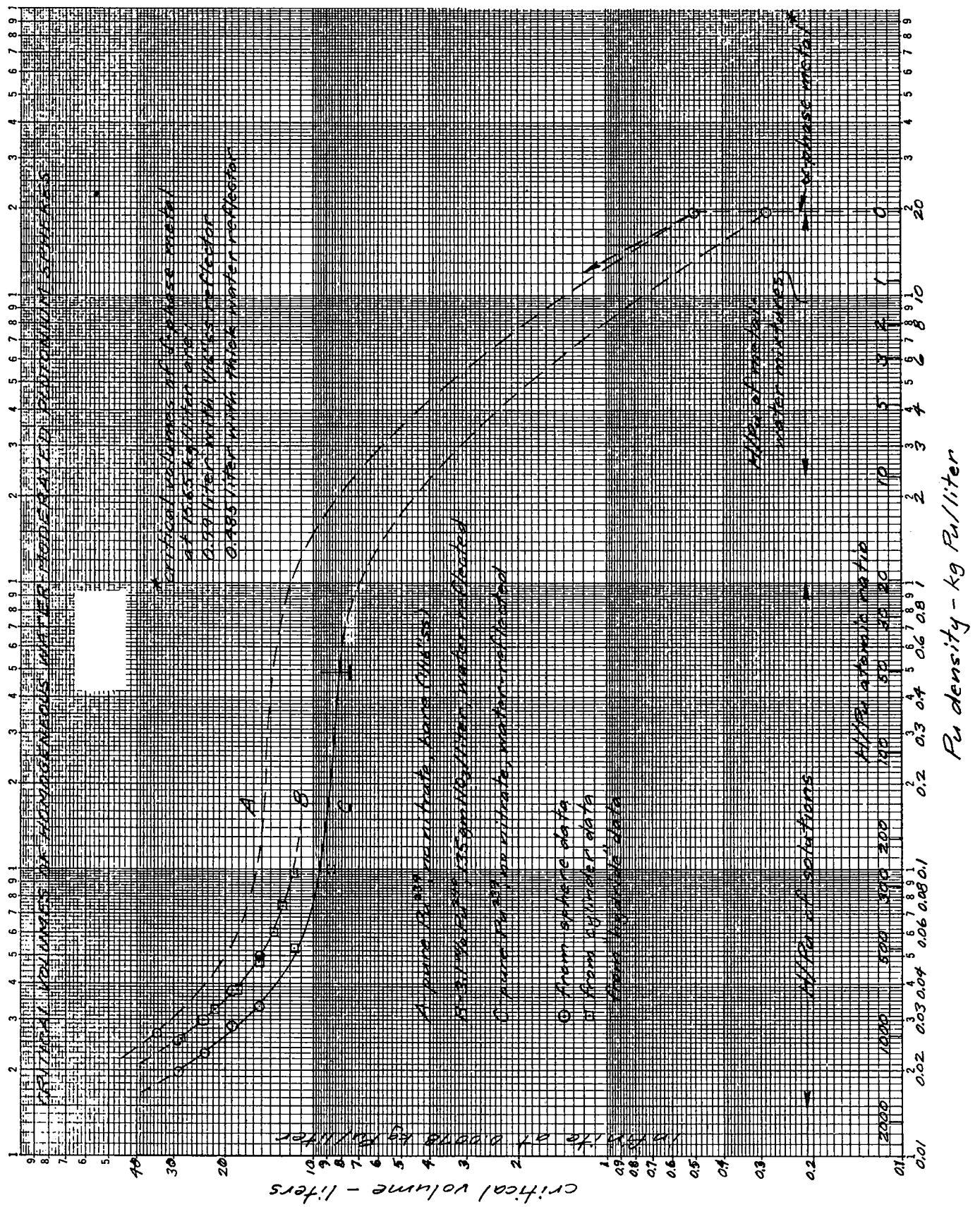


Fig. 6.

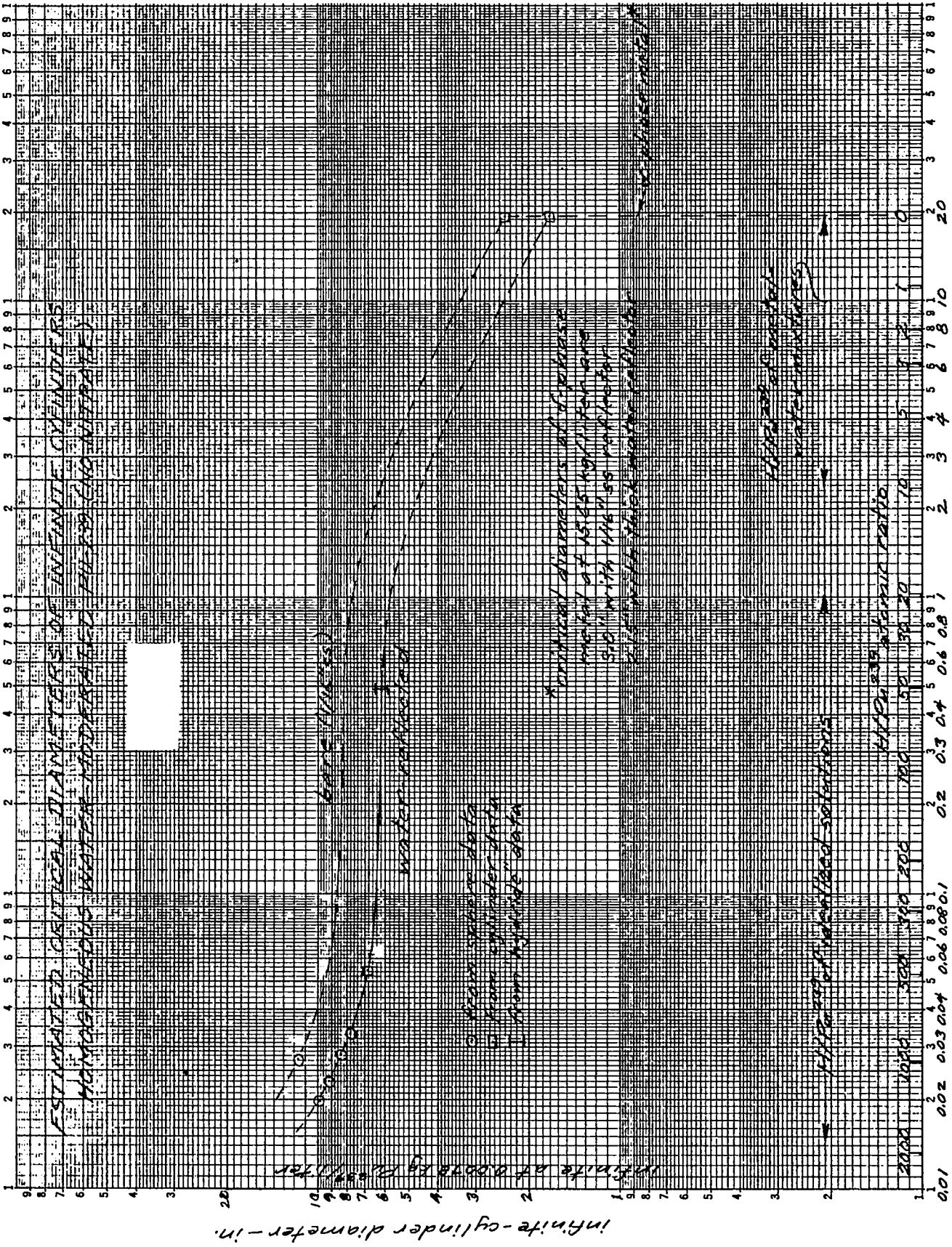


Fig. 7. - 14 -

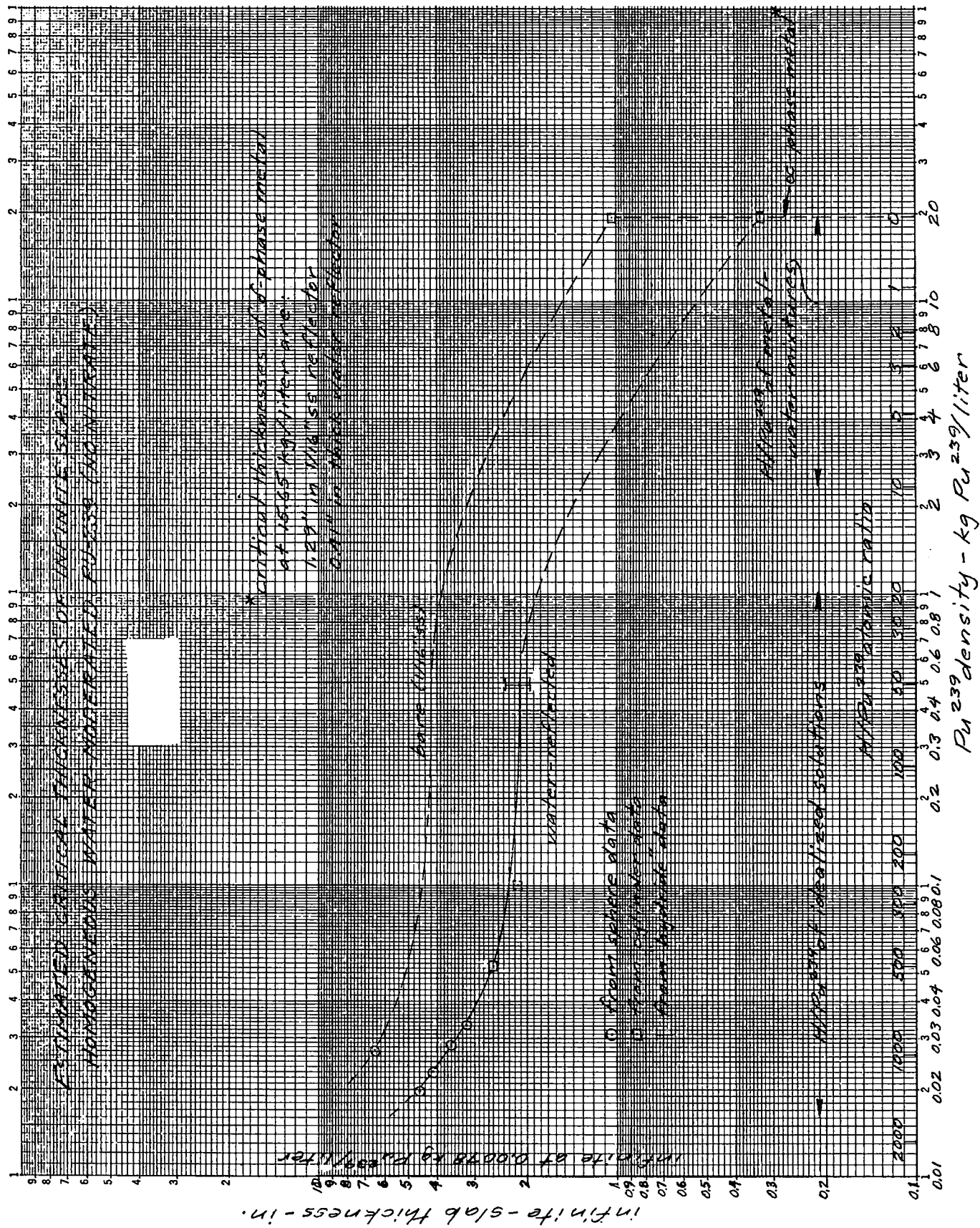
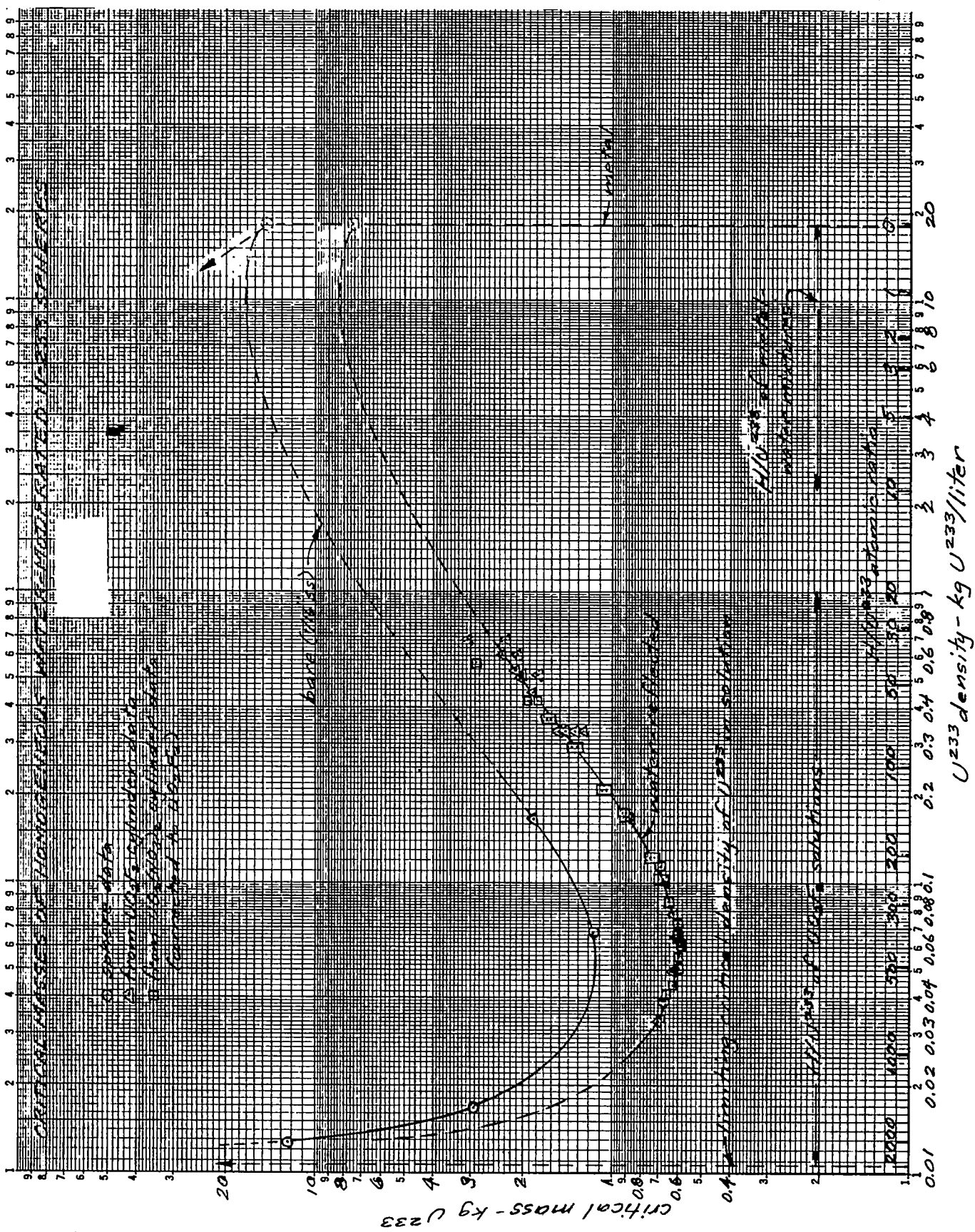


Fig. 8. - 15 -

Fig. 9. - 16 -



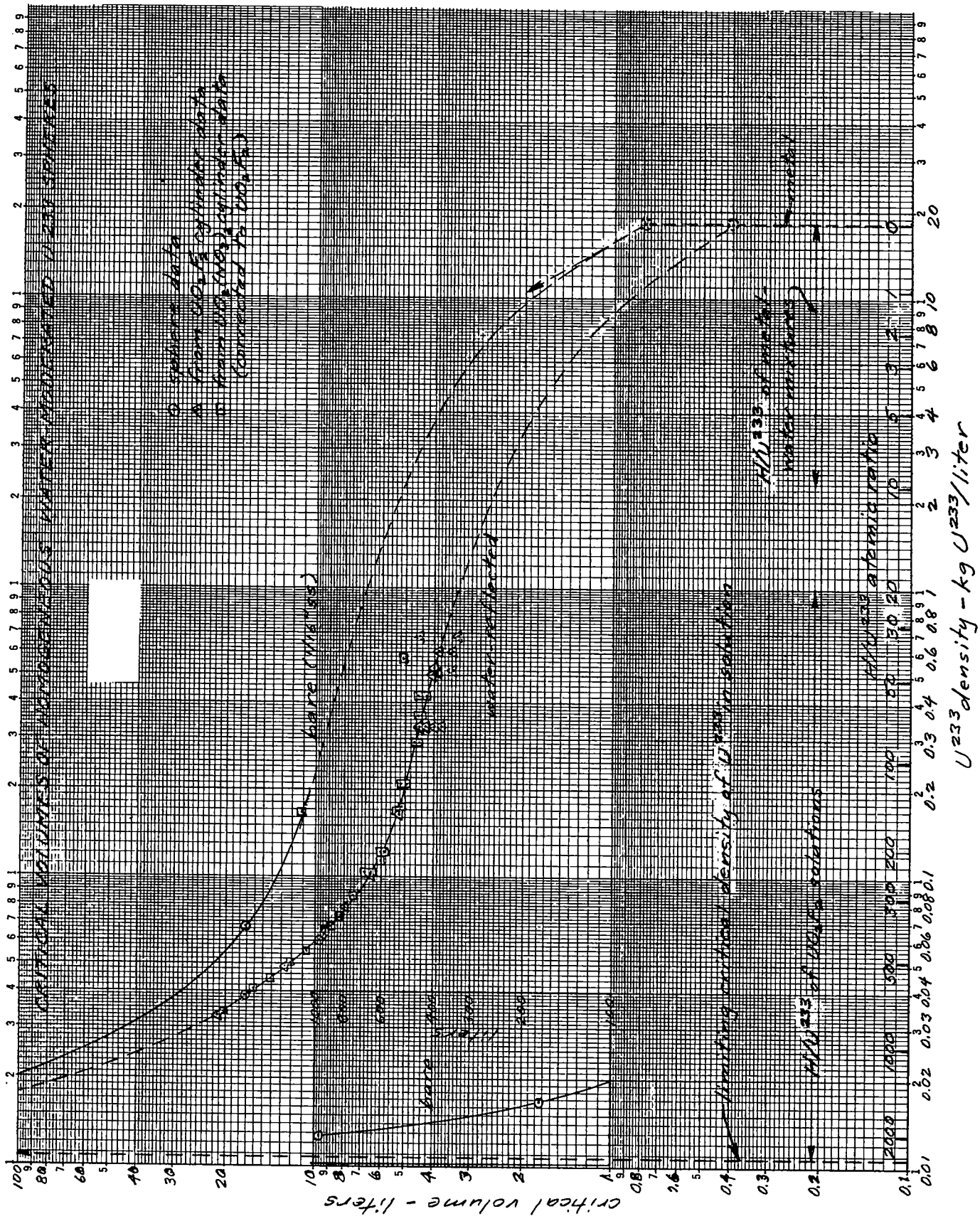
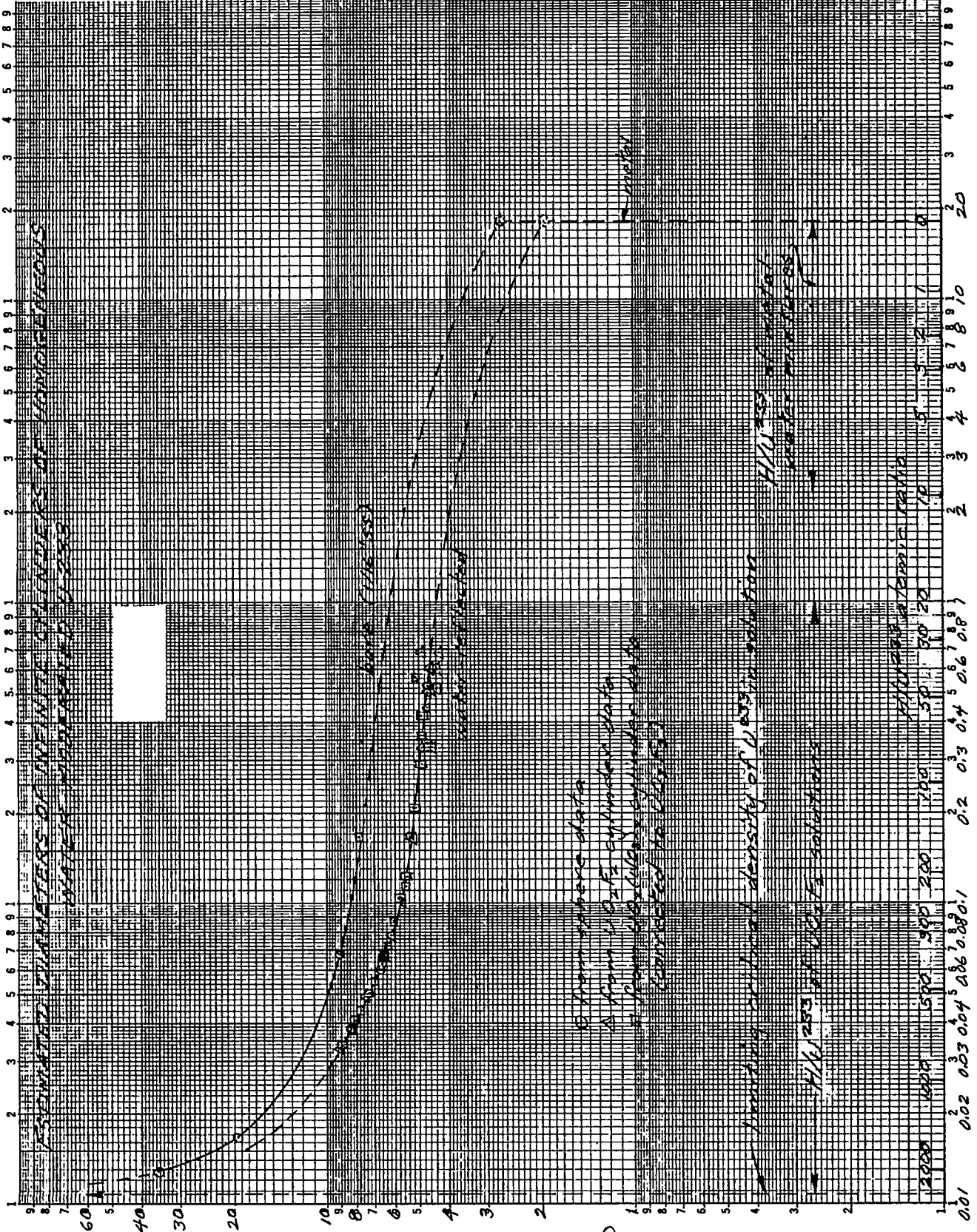


Fig. 10.



infinite-cylinder diameter - in.

Regions of instability of homogeneous mixture of water & U-233

Regions of instability of U-233 solution

Regions of instability of U-233 solution

From infinite data
 A from 1/2 ft diameter data
 From 1/8 inch diameter data corrected to 1/2 ft

H₂O density - kg U²³³/liter

U ²³³ density	U-233 density
0.01	0.02
0.02	0.03
0.04	0.04
0.06	0.06
0.1	0.1
0.2	0.2
0.3	0.3
0.4	0.4
0.6	0.6
1.0	1.0
2.0	2.0
3.0	3.0
4.0	4.0
5.0	5.0
6.0	6.0
7.0	7.0
8.0	8.0
10.0	10.0
15.0	15.0
20.0	20.0
30.0	30.0
40.0	40.0
50.0	50.0
60.0	60.0
80.0	80.0

Fig. 11. - 18 -

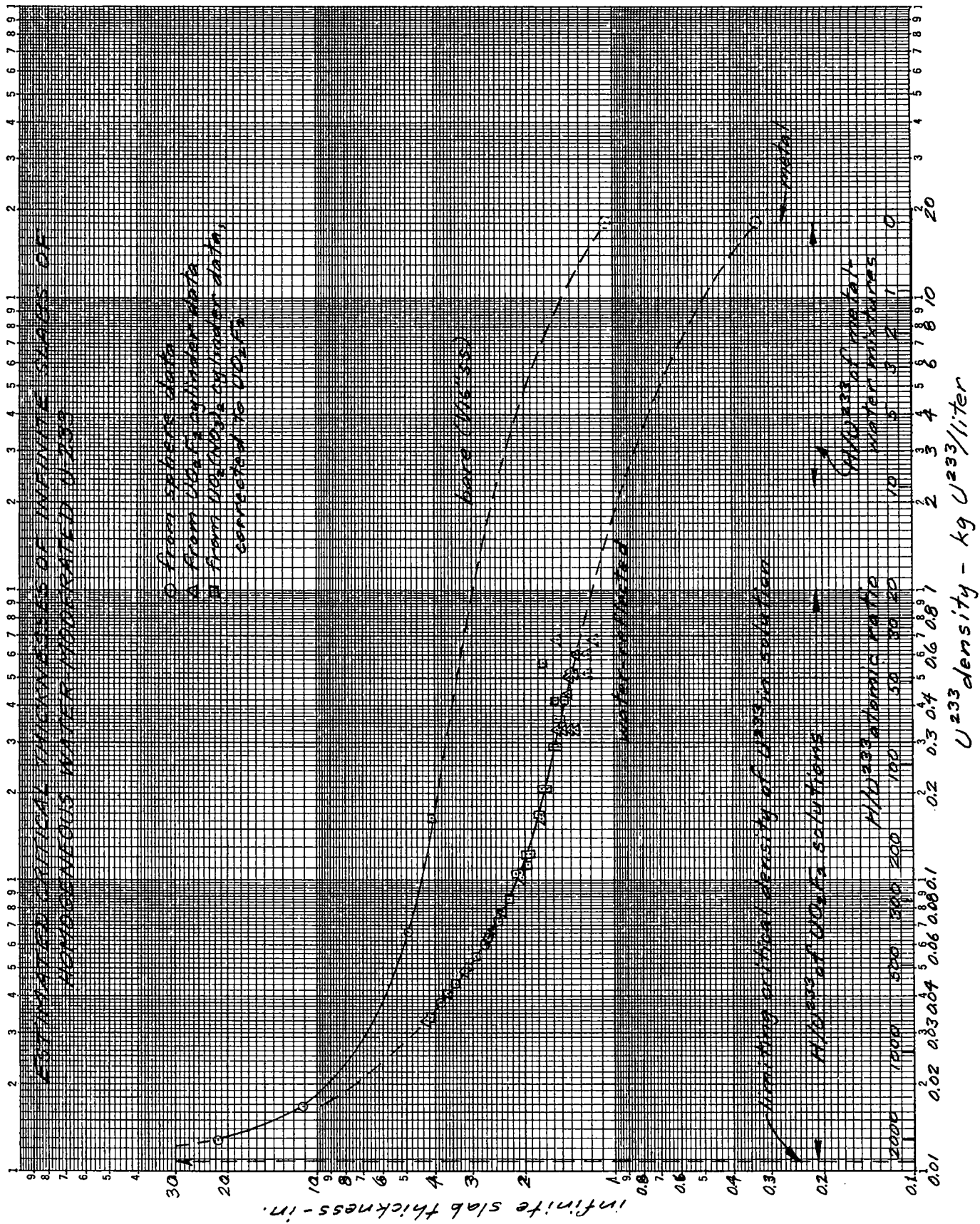
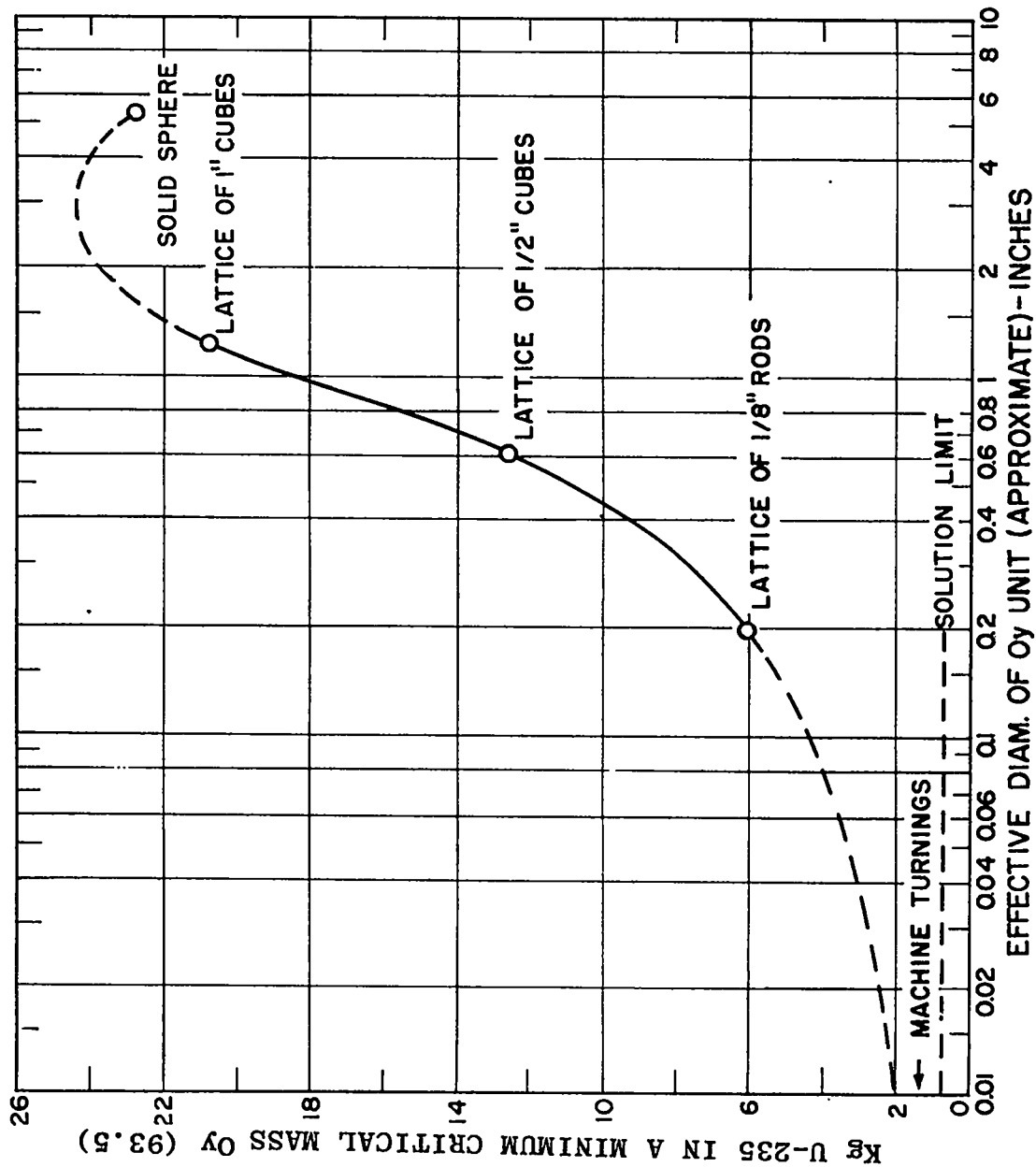


Fig. 12. - 19 -



Minimum critical mass of flooded Oy (93.5) metal lattices as a function of or alloy unit size.

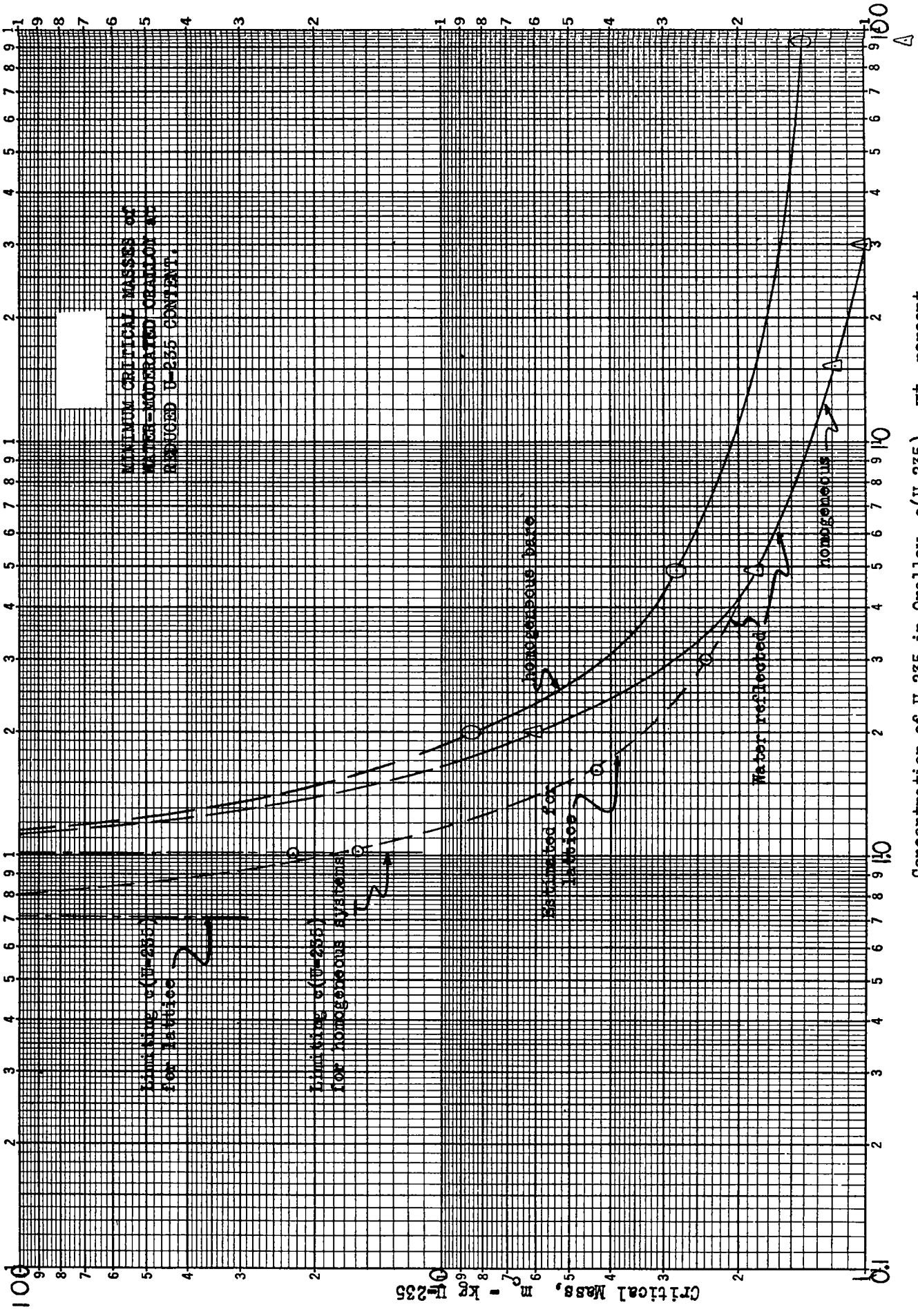


Fig. 14a. - 21 -

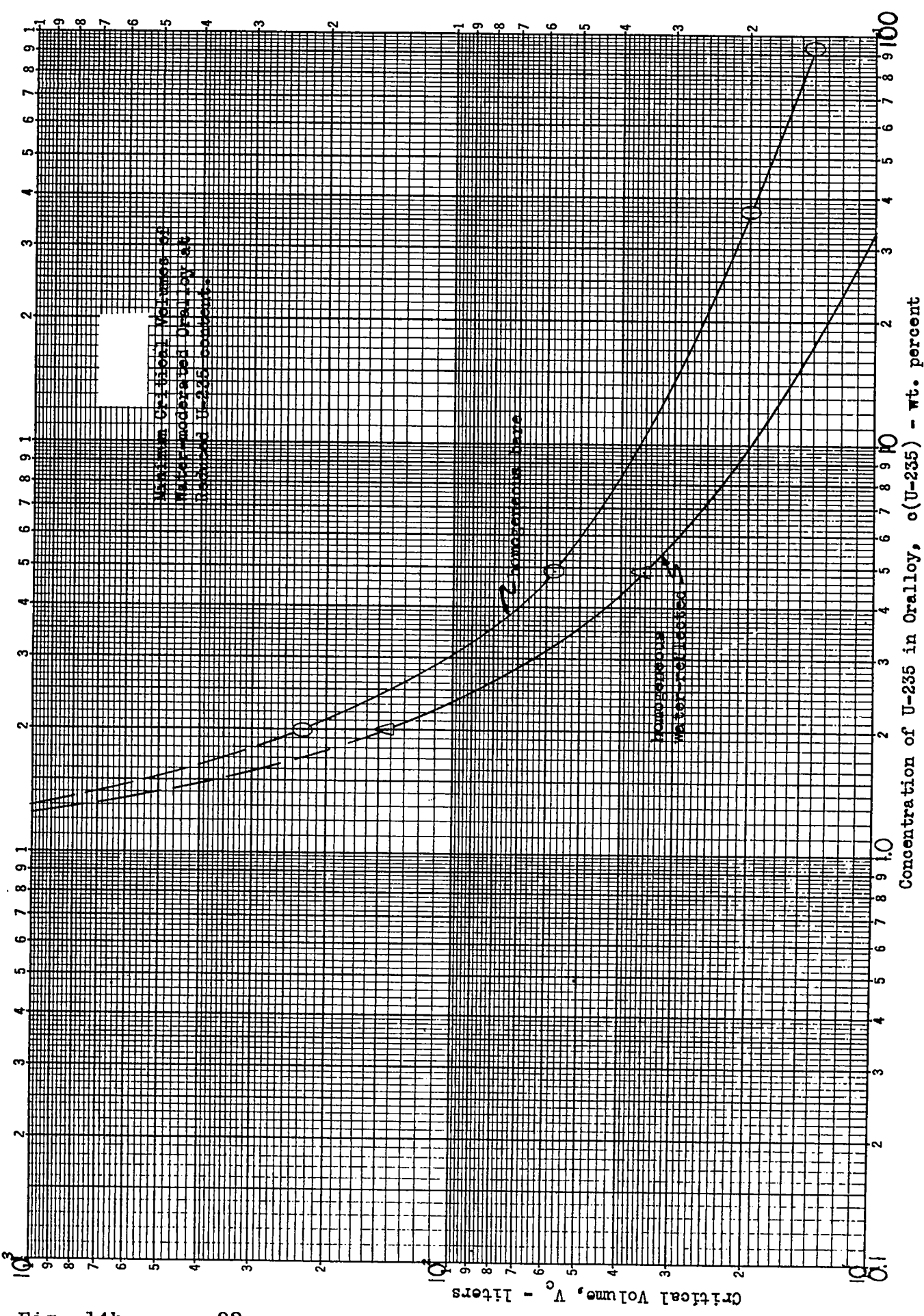
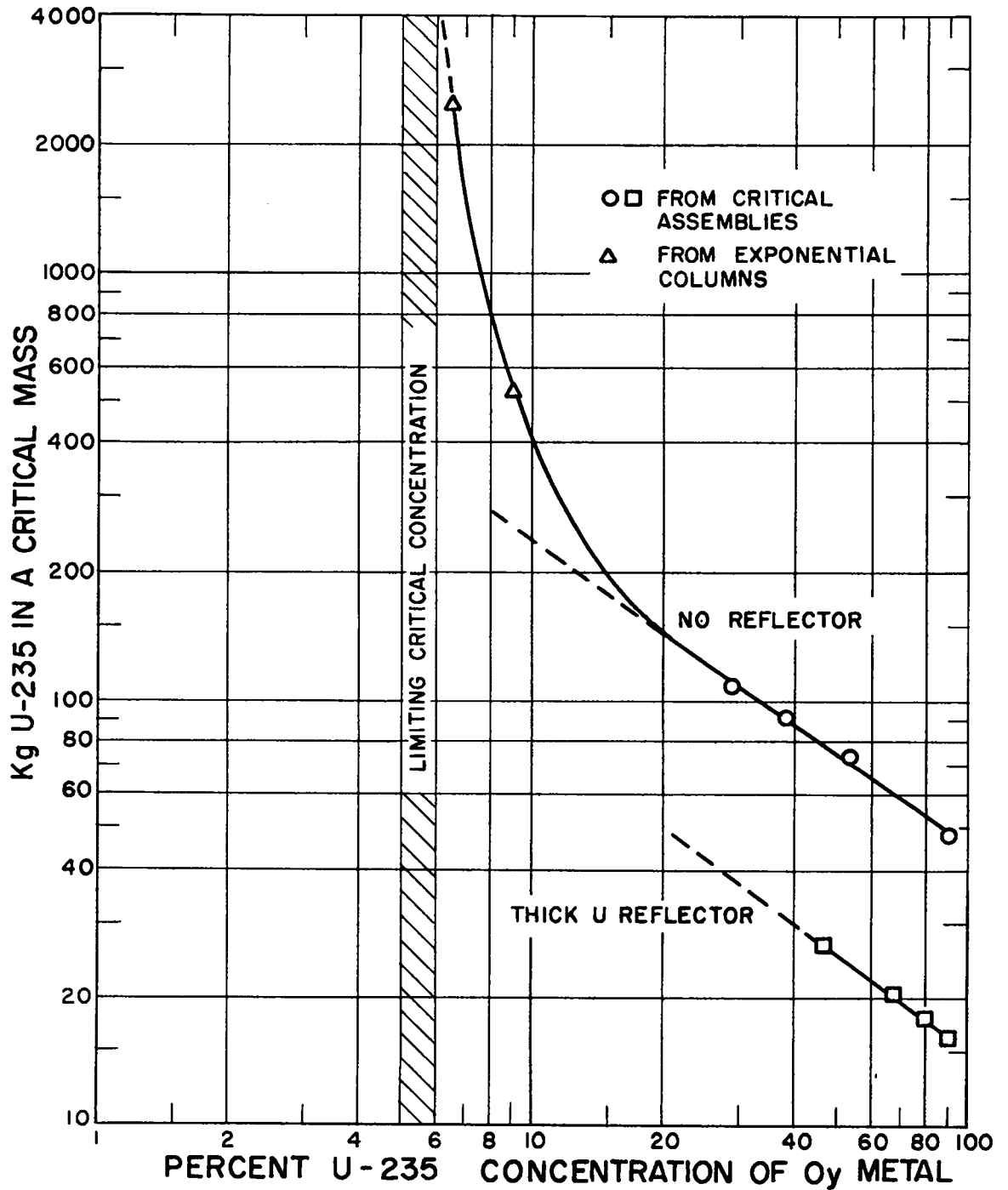


Fig. 14b. - 22 -

A



Critical mass vs. U-235 concentration of oralloy metal. (The shaded strip represents the range of uncertainty in the value of U-235 concentration below which oralloy metal cannot be made critical.)

Poisoned solutions

The influence of excess nitrate on critical mass of water-reflected Pu²³⁹ solutions is presented in Figure 16.⁽⁷⁾ Observations on effects of heterogeneous poisons in U²³⁵ solutions are summarized in Table I⁽⁴⁰⁻⁴³⁾ and Figure 17. The figure shows the influence of various degrees of Pyrex poisoning on the critical height of 20"-diam. aqueous solutions of U²³⁵, both bare and water-reflected. The Raschig rings with which one point was obtained were 2.375" OD x 2" ID x 2.375" long and were packed randomly throughout the solution volume.

Data from the Physical Constants Testing Reactor⁽³⁶⁾ establish the quantity of uniformly-distributed boron that is required to reduce to unity the k_{∞} of a fissionable mixture. For Oy(3.04% U²³⁵)O₃ - polyethylene mixtures, 0.37 atom B per atom of U²³⁵ (17 gm B/kg U²³⁵) protects against criticality for the entire range of H/U²³⁵; at H/U²³⁵ = 1430, k_{∞} = 1 without boron. In the case of an Oy(2% U²³⁵)F₄ - paraffin mixture at H/U²³⁵ = 195, 0.25 atom B per atom of U²³⁵ gives k_{∞} = 1, from which it is estimated that 0.26 atom B per atom of U²³⁵ (~ 12 gm B/kg U²³⁵) protects for all H/U²³⁵.

Systems with nonhydrogenous diluents

Some effective cross sections from reactivity coefficient data and resulting dilution exponents for bare Oy(94) (Godiva), Oy(94) in an 8-1/2"-thick U reflector (Topsy), and bare Pu (Jezebel) are listed in Table II.⁽⁴⁴⁾ In terms of the dilution exponent $n(x)$ for the material x , the critical mass of fissionable material diluted homogeneously with the volume fraction F of the material x is

$$m_c = m_{c_0} (1-F)^{-n}, \quad F \ll 1,$$

where m_{c_0} is the critical mass of the undiluted system. In the cases of D₂O, graphite ($\rho_0 = 1.67 \text{ gm/cm}^3$) and BeO ($\rho_0 = 2.86 \text{ gm/cm}^3$) diluting unreflected Oy(~ 93),⁽⁴⁵⁻⁴⁷⁾ data exist over an extended range

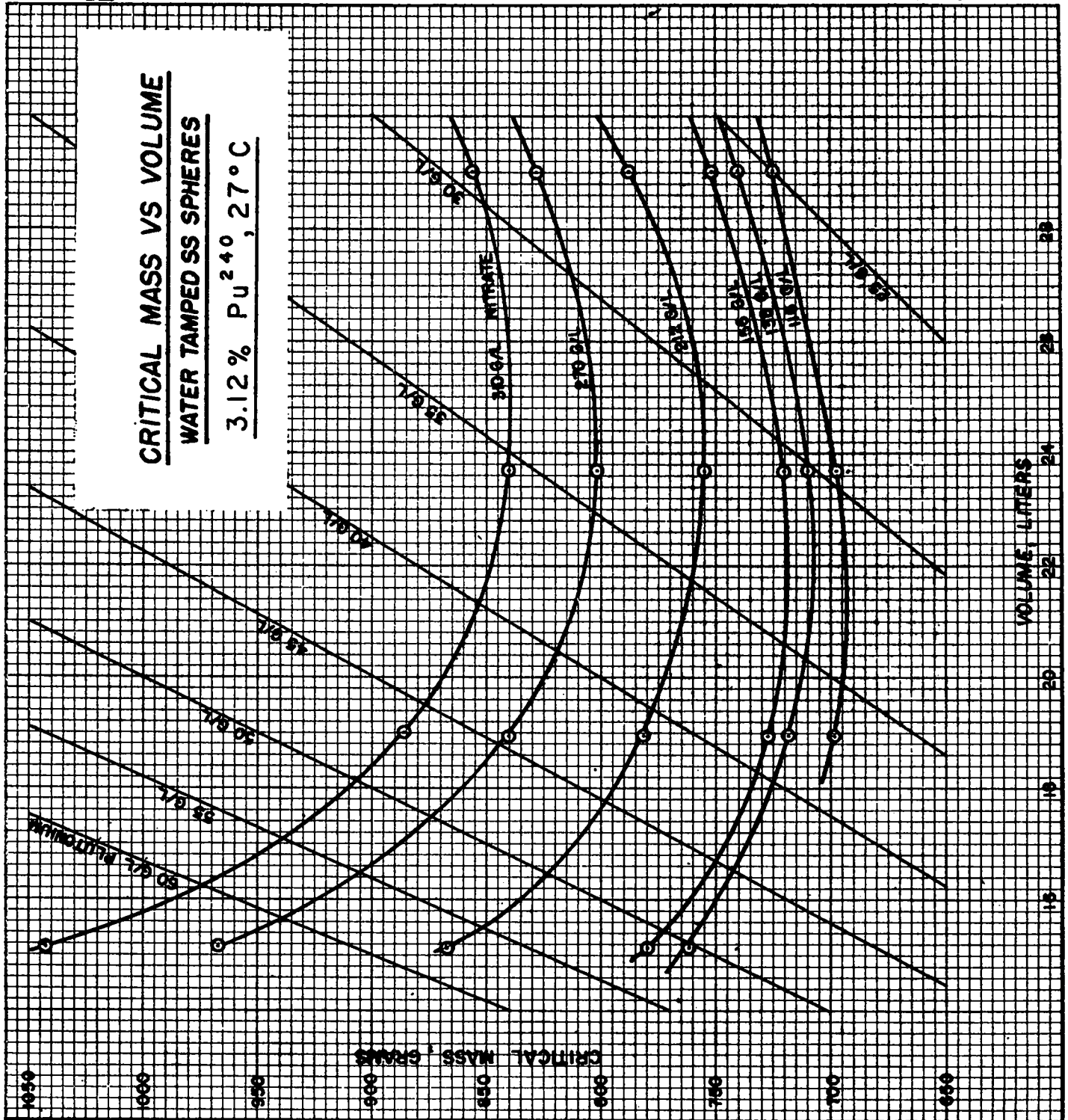
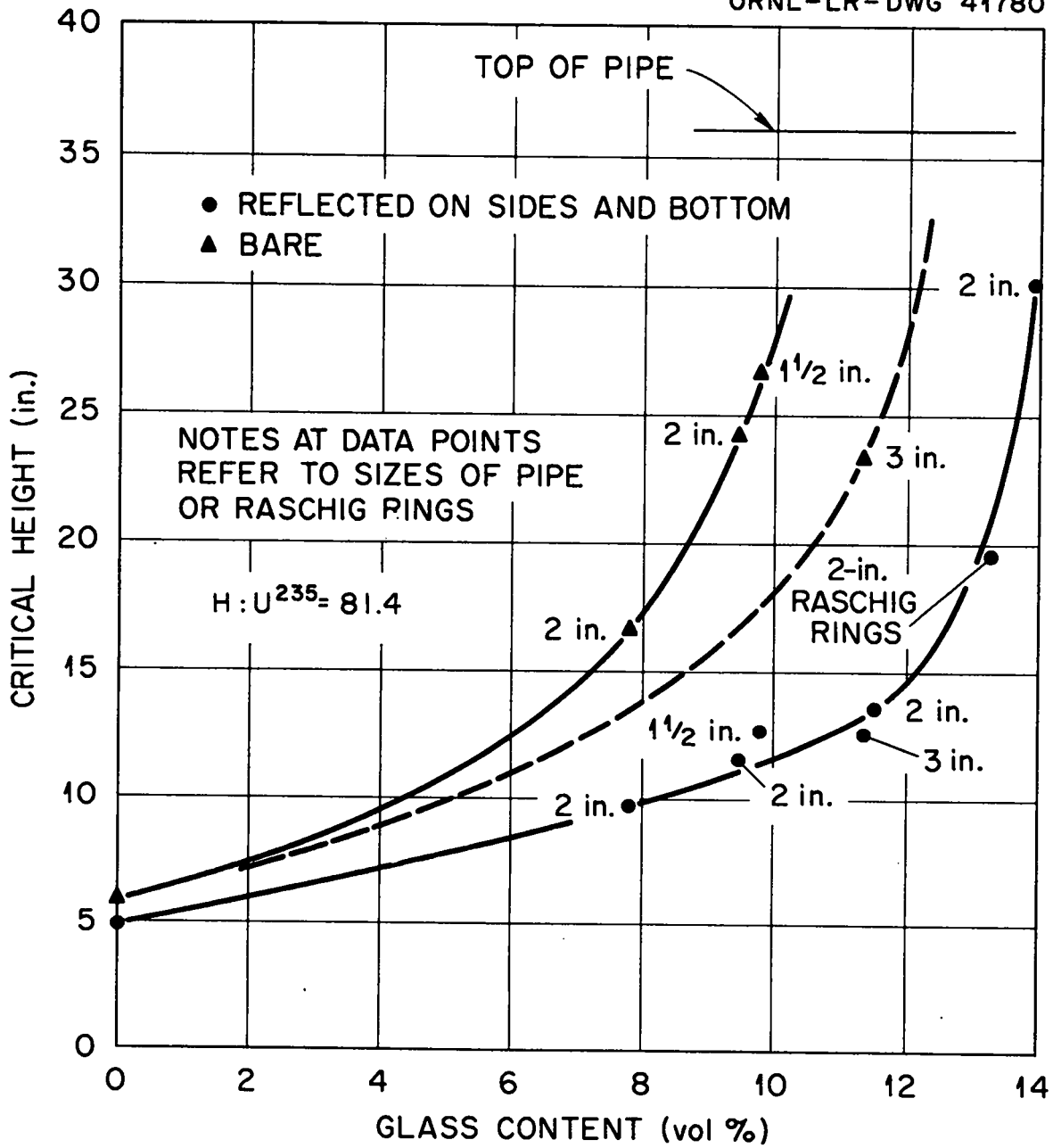


Fig. 16. - 25 -

TABLE I.
 U^{235} SOLUTIONS WITH HETEROGENEOUS POISONS

<u>Container</u>	<u>Solution</u>	<u>Reflector</u>	<u>Poison</u>	<u>Critical Height</u>
<u>Oy (~ 93):</u>				
15" diam. ss cylinder	OyO_2F_2 $H/U^{235} = 73.0$	water	136 steel rods, 7/8"diam. (49.2v/o of core)	37.5"
30" x 60" Al tank	OyO_2F_2 $H/U^{235} = 78.7$	water (half-reflected)	10 boral partitions 2.3" wide (3/8" boral, $\sim 0.3 \text{ gmB/cm}^3$)	6.9"
10" diam. Al cylinder in 1/4"-thick Cu	OyO_2F_2 $H/U^{235} = 52.6$	water outside 1/4" Cu	33.7v/o Cu ~ 0.15 " thick, min. spacing $\sim 3/4$ "	60"
42" diam. ss tank	$OyO_2(NO_3)_2$ $\leq 360 \text{ gm } U^{235}/\text{liter}$	concrete (on sides)	random-packed. Pyrex raschig rings, 1.5"ODx1.5"highx7/64"wall (17.8 v/o Pyrex containing 12-1/2 w/o B_2O_3)	subcritical at 460 liters solution
<u>Oy (~ 87):</u>				
20" diam. Al cylinder	$OyO_2(NO_3)_2$ $H/U^{235} = 81.4$	water on sides, bottom	Pyrex tubing, or rings ≤ 2 " ID (~ 4 w/o B): 7.8 v/o glass 9.45 v/o glass 11.5 v/o glass 13.3 v/o glass 13.95 v/o glass 16.7 v/o glass	9.75" 11.6" 13.6" 19.6" 30.1" subcritical at 36" depth
"	same except $H/U^{235} = 141$	"	same, 7.8 v/o glass	12.5"
"	same except $H/U^{235} = 276$	"	same, 7.8 v/o glass	subcritical at 36" depth



CRITICAL HEIGHTS of 20-in-diam STAINLESS STEEL CYLINDERS
CONTAINING PYREX-POISONED SOLUTIONS of $Oy(87.4)O_2 (NO_3)_2$

TABLE II.
SELECTED MATERIAL REPLACEMENT RESULTS FOR TOPSY, GODIVA, AND JEZEBEL

Element (x)	Density gm-atom/cm ³	Topsy (Oy 94% in U)			Godiva (bare Oy 94%)			Jezebel (bare Pu)		
		$\bar{\sigma}_a(x)^a$ barn	$\bar{\sigma}_{tr}(x)$ barn	Dilution exponent ^b n(x)	$\bar{\sigma}_a(x)^a$ barn	$\bar{\sigma}_{tr}(x)$ barn	Dilution exponent ^b n(x)	$\bar{\sigma}_a(x)^a$ barn	$\bar{\sigma}_{tr}(x)$ barn	Dilution exponent ^b n(x)
C	0.185	-0.022	2.13	0.86	-0.028	2.17	1.02	0.016	2.15	1.30
O		-0.013	2.20					0.023	2.22	
Al	0.100	-0.006	2.12	1.04	-0.006	2.14	1.51	0.033	2.30	1.61
Cr	0.138	0.015	2.41	0.98						
Mn	0.135	0.009	2.70	0.95						
Fe	0.137	0.020	2.29	1.01	0.006	2.29	1.28	0.050	2.44	1.45
Ni	0.152	0.066	2.77	1.02	0.056	2.65	1.22	0.111	2.77	1.39
Cu	0.141	0.035	2.68	0.99	0.022	2.73	1.18	0.074	2.83	1.37
Zr	0.071	0.022	3.87	1.02				0.070	4.10	1.51
Nb	0.092	0.068	3.99	1.01						
Mo	0.106	0.032	4.58	0.89				0.105	3.99	1.33
Ta	0.092	0.155	3.91	1.12				0.232	4.34	1.48
W	0.105	0.097	4.40	0.99				0.182	4.60	1.30
Th	0.049 ₅	0.069	4.48	1.08	0.017	4.92	1.46	0.141	5.00	1.66
U ²³³	0.080	-3.22 ₀								
U ²³⁵	0.080	-1.89 ₃ ^c			-1.86 ₀ ^c			-1.82 ₈	5.3	
U ²³⁸	0.080	-0.228	5.10 ^c		-0.299	5.0 ^c		-0.238	5.1 ^a	
Pu ²³⁹		-3.63 ₆			-3.56 ₁			-3.60 ₀ ^a	5.3	
Pu ²⁴⁰		-2.58						-2.34		
Void				1.20			2.00			2.00

(Footnotes on next page)

TABLE II Footnotes

^a $\bar{\sigma}_a(x) = \bar{\sigma}_c(x) - \bar{\sigma}_f(x) - \Delta\gamma\bar{\sigma}_s(x)$, where $\bar{\sigma}_c$ and $\bar{\sigma}_f$ are capture and fission cross-sections (suitably averaged), $\Delta\gamma$ is the increase in neutron effectiveness per central scattering and $\bar{\sigma}_s$ is scattering cross section.

^b The critical mass of a system diluted by the volume fraction $F(x)$ of element x , $m_c(x)$, is related to the critical mass of the undiluted system $m_c(o)$, according to $m_c(x)/m_c(o) = [1-F(x)]^{-n}$; if $F(x) \ll 1$. Where $\rho_o(x)$ is the normal density of x in gm-atom/cm³,

$$n(x) = 1.20 - \rho_o(x) [0.735 \bar{\sigma}_{tr}(x) - 12.82 \bar{\sigma}_a(x)], \text{ for Topsy;}$$

$$n(x) = 2.00 - \rho_o(x) [2.25 \bar{\sigma}_{tr}(x) - 14.27 \bar{\sigma}_a(x)], \text{ for Godiva;}$$

$$n(x) = 2.00 - \rho_o(x) [1.846 \bar{\sigma}_{tr}(x) - 9.964 \bar{\sigma}_a(x)], \text{ for Jezebel.}$$

^c These values are used for normalization.

(Figure 18). Figure 19 gives critical masses of bare and U-reflected cylinders of Pu diluted by Al, Fe, U, and Th. (48)

Systems at reduced density

The dependence of critical mass (m_c) upon core density (ρ) has been determined for several spheres or nearly equilateral cylinders. (6,13)

Values of n in the relation $m_c = \text{const } (\rho/\rho_0)^{-n}$ are

1.20 for Oy(94) metal in 8-1/2" U reflector

1.57 for Oy(93) H_3C in 8" thick U reflector

1.88 for Oy(93) O_2F_2 solution at $H/U^{235} = 230$ in thick water reflector (possibly influenced by void geometry)

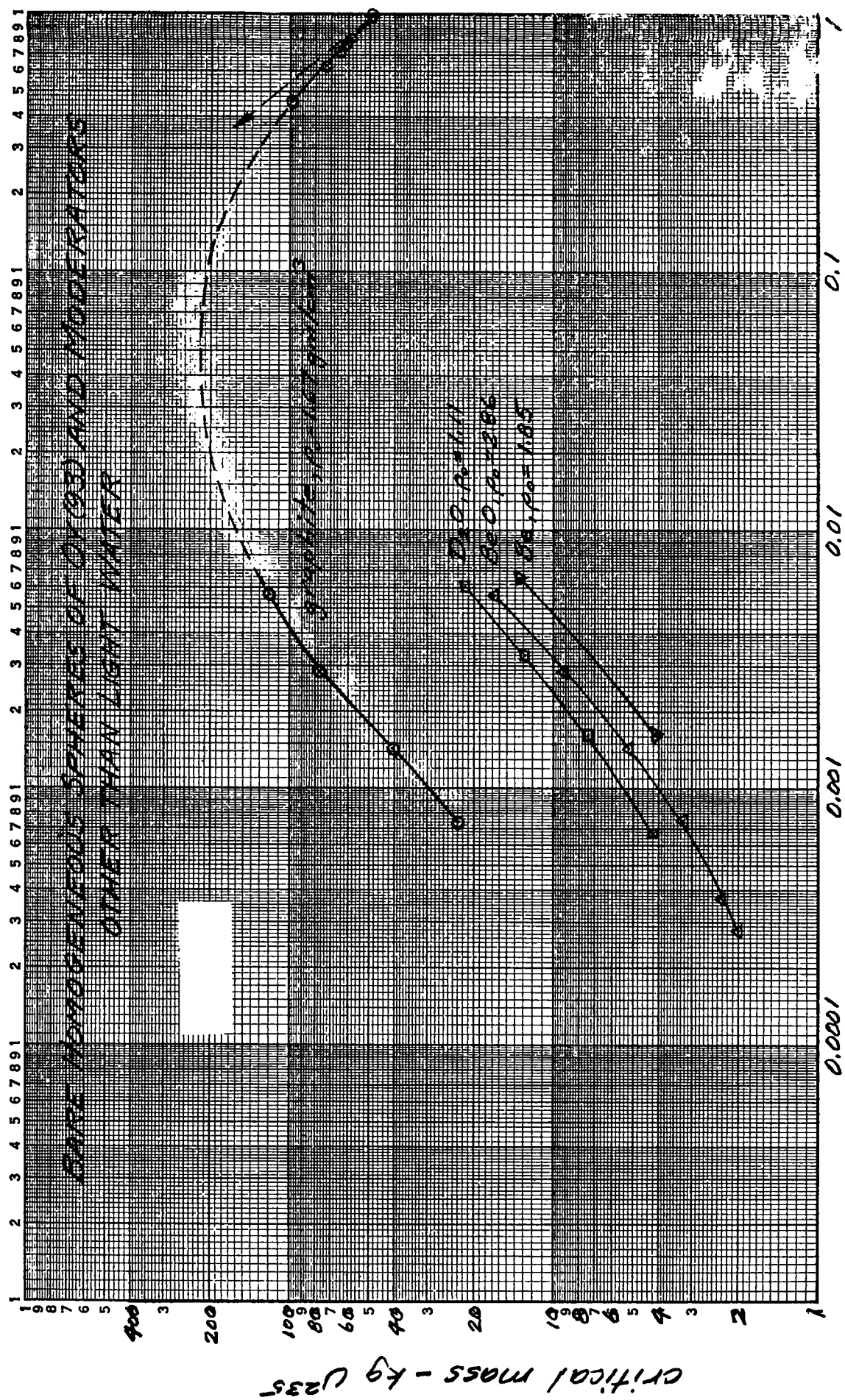
~ 1.1 for Pu^{239} metal in a reflector corresponding to thick U (from Figure 22)

Where density of both core and reflector of a spherical system are changed by the ratio ρ/ρ_0 , and the ratio of reflector thickness to core radius is maintained, then $n = 2$ (the value for an unreflected spherical core).

In the case of an infinite slab, the critical mass per unit area is necessarily independent of ρ .

Spherical systems with various reflectors

Critical masses of unmoderated Oy (93.5) metal spheres are given for various reflectors as functions of reflector thickness in Figures 20 and 21, with supplementary data in Table III. (6) Figure 22 gives critical masses of U^{233} metal, δ -phase Pu^{239} and α -phase Pu^{239} in terms of the critical mass of Oy (93.5) metal in a reflector of the same composition and thickness. (34,49-51) As the existing data show no distinction between nonmoderating and moderating reflectors (of limited thickness), these curves provide a basis for estimating critical masses of the other materials from the abundant data for Oy.



volume fraction of Oy (93) metal at 18.8 gm/cm³

critical mass - kg U₂₃₅

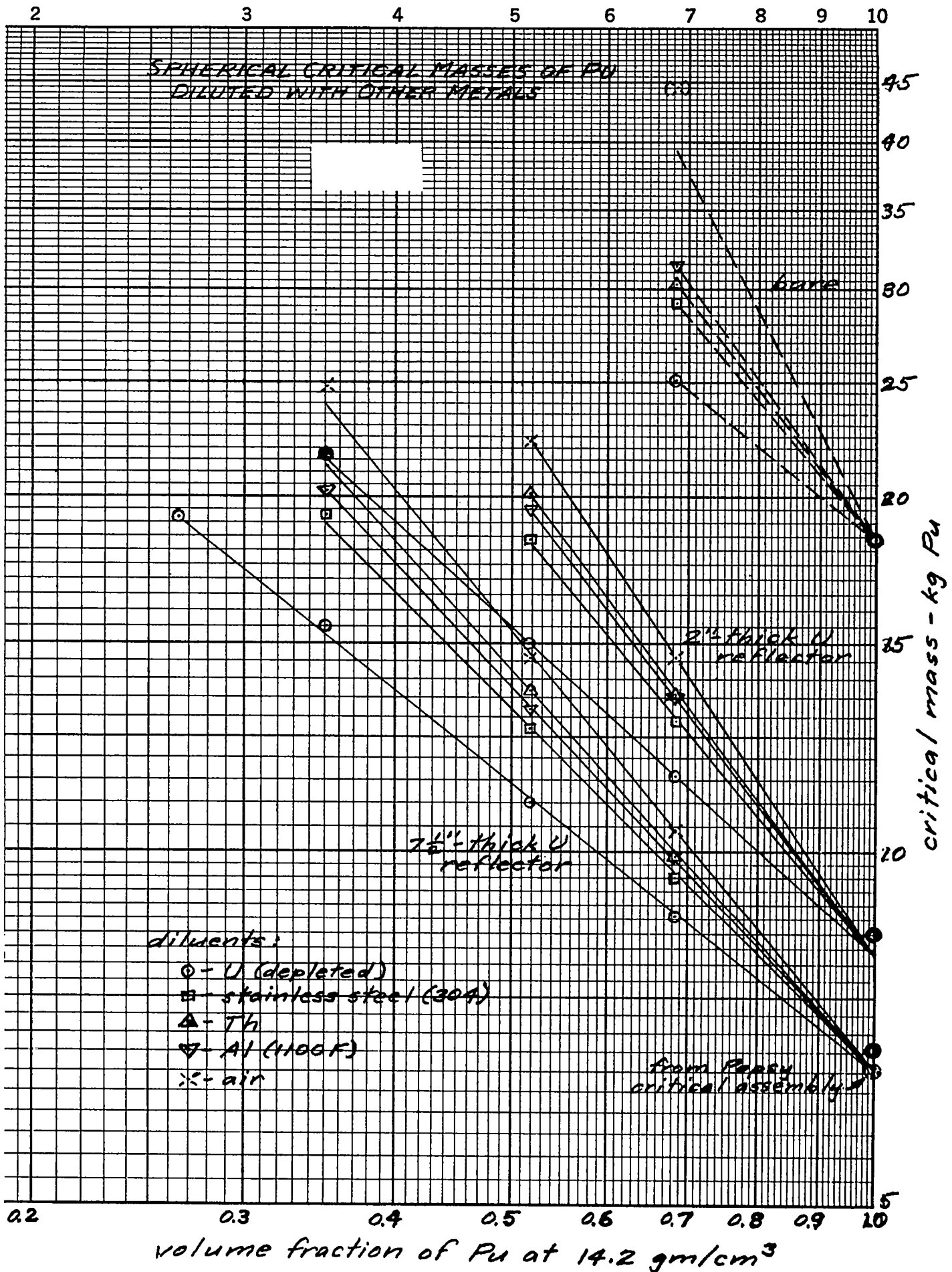
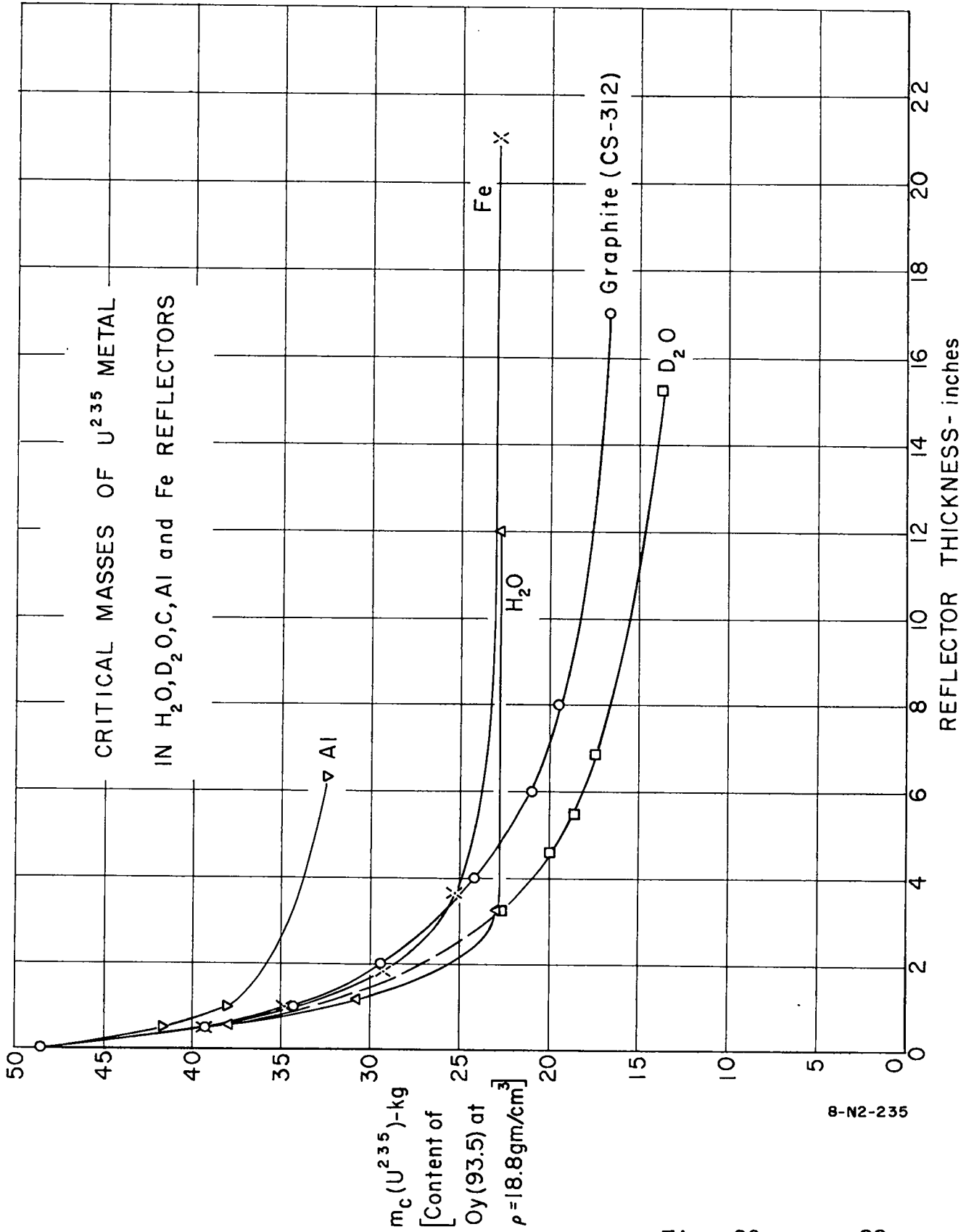
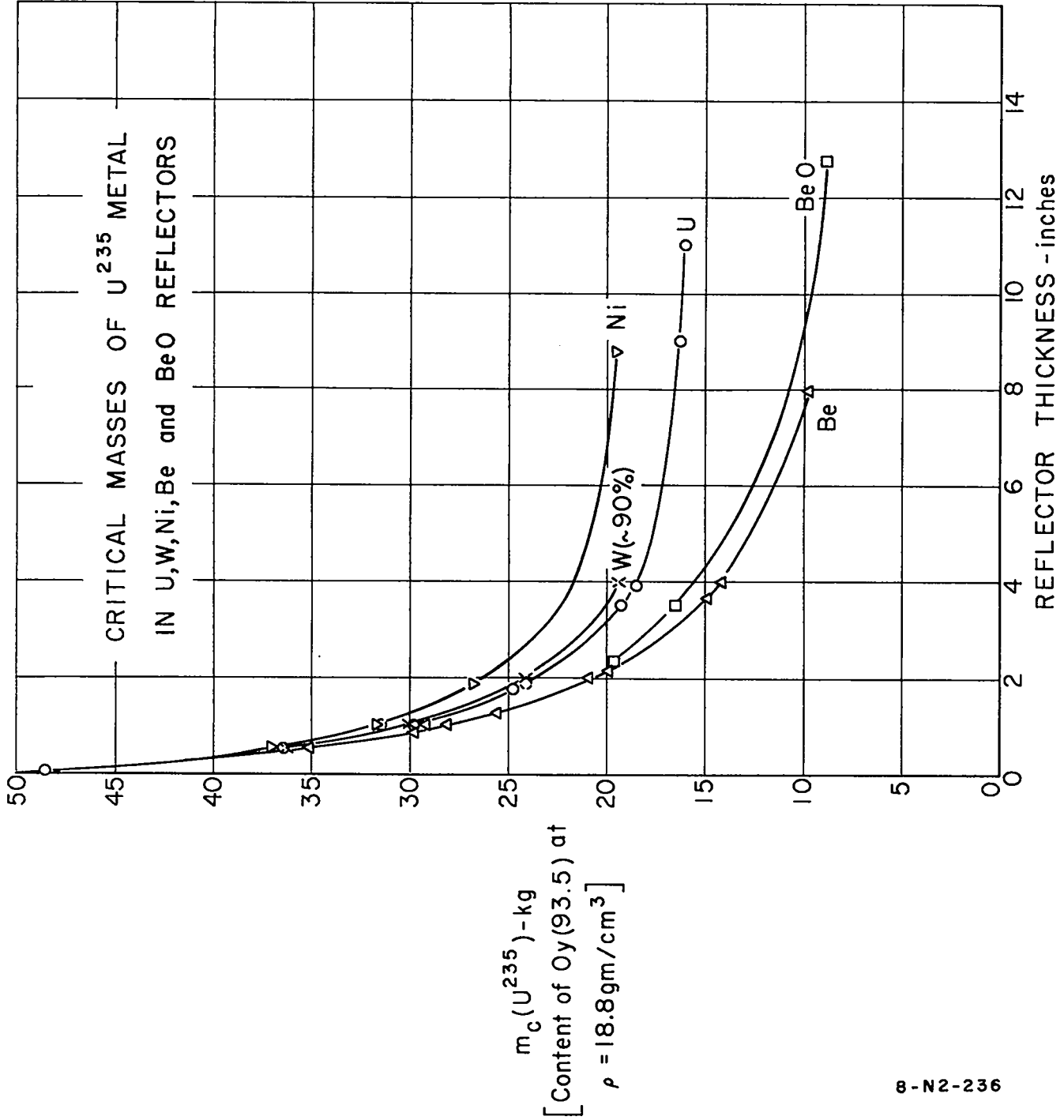


Fig. 19.



8-N2-235

Fig. 20. - 33 -



8-N2-236

Fig. 21. - 34 -

TABLE III.
CRITICAL MASSES OF SPHERICAL ORALLOY (93.5 w/o U-235) WITH VARIOUS REFLECTORS

Reflector (ρ -gm/cm ³)	Data Adjusted to the Following Standard Reflector Thicknesses.				Effective σ_{tr} -cm ⁻¹
	1 in. Critical mass - kg	2 in. U-235 at	4 in. $\rho(Oy)=18.8$ gm/cm ³	infinite	
Be (QMV, $\rho = 1.84$)	29.2	20.8	14.1		~0.25
BeO ($\rho = 2.69$)		21.3	15.5	~ 8.9	
WC ($\rho = 14.7$)		21.3	16.5	~16.0	
U ($\rho = 19.0$)	30.8	23.5	18.4	16.1	0.25
W-alloy (~92% W, $\rho=17.4$)	31.2	24.1	19.4		~0.25
Paraffin	(32.6)			21.8	
H ₂ O	(33.5)	~24.0	22.9	22.8	
D ₂ O		(27)	21.0	~13.6	
Cu ($\rho = 8.88$)	32.4	25.4	20.7		0.23
Ni ($\rho = 8.88$)	33.0	25.7	(21.5)	19.6	0.23
Al ₂ O ₃ ($\rho = 2.76$)	35.1				
Graphite (CS-312, $\rho=1.69$)	35.5	29.5	24.2	~16.7	0.18
Fe ($\rho = 7.87$)	36.0	29.3	25.3	23.2	0.19
Zn ($\rho = 7.04$)		29.8	25.0		0.18
Th ($\rho = 11.48$)		33.3			~0.14
Al (2S, $\rho = 2.70$)	39.3	(35.5)	(32)	<30.0	0.13
Ti ($\rho = 4.50$)	39.7				0.12
Mg ($\rho = 1.77$)	41.0				0.10

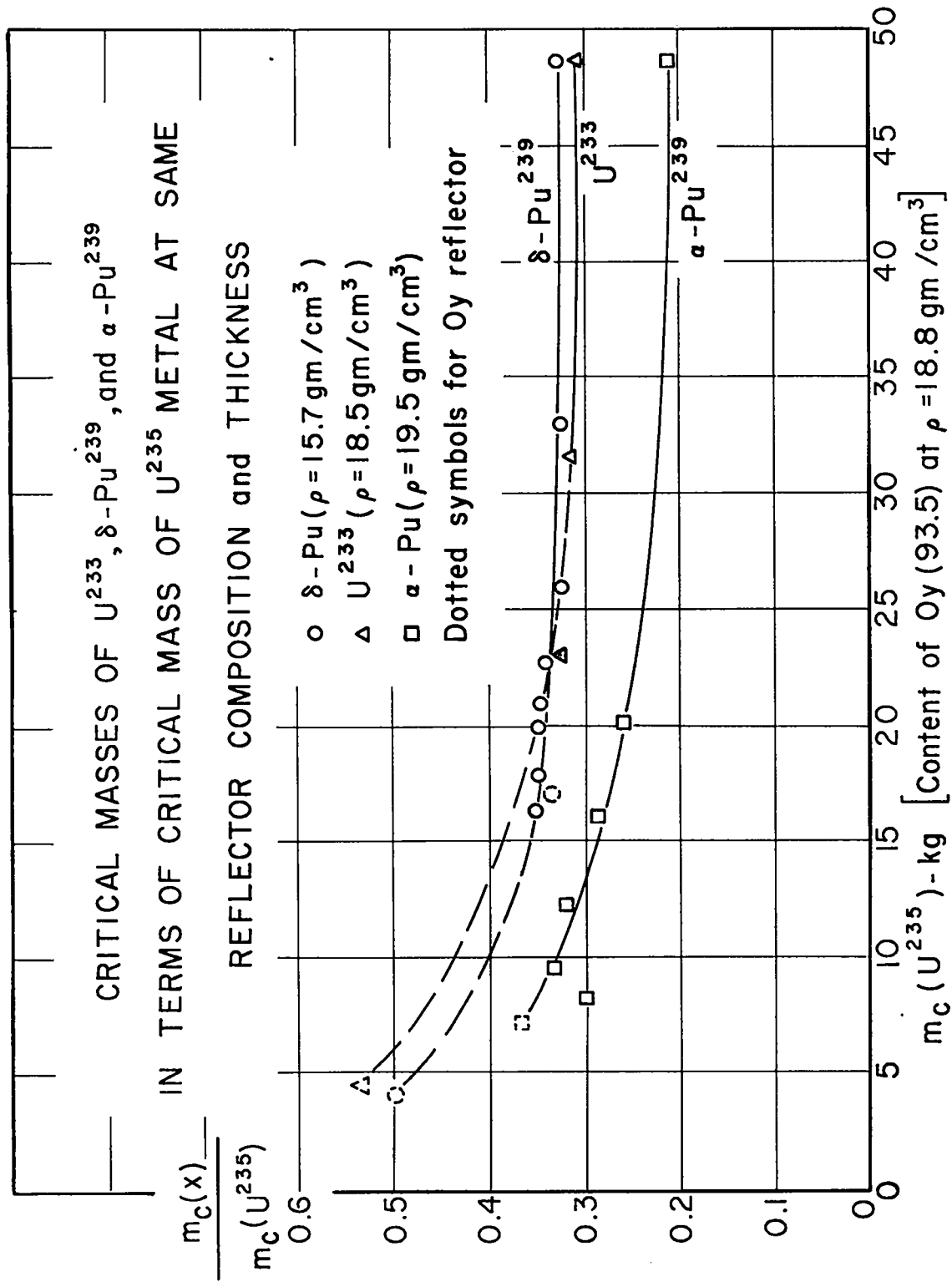


Fig. 22.

No such range of reflector data exists for solutions. It has been observed that the same thickness of iron is essentially equivalent to the inner two inches (or less) of a thick water reflector about U^{235} solution. (13) Similar replacements show that plexiglas is a slightly more effective reflector than water. Figure 23 shows critical height of a 10"-diameter U^{235} solution (0.337 kg U^{235} /liter) as a function of thickness of lateral water reflector and of lateral furfural reflector. (5) The critical height of a slab of U^{235} solution (0.483 kg U^{235} /liter), 4' wide x 6" thick, vs. thickness of Al reflector on each face is given by Figure 24. (11)

Cylinders of various height/diameter ratios

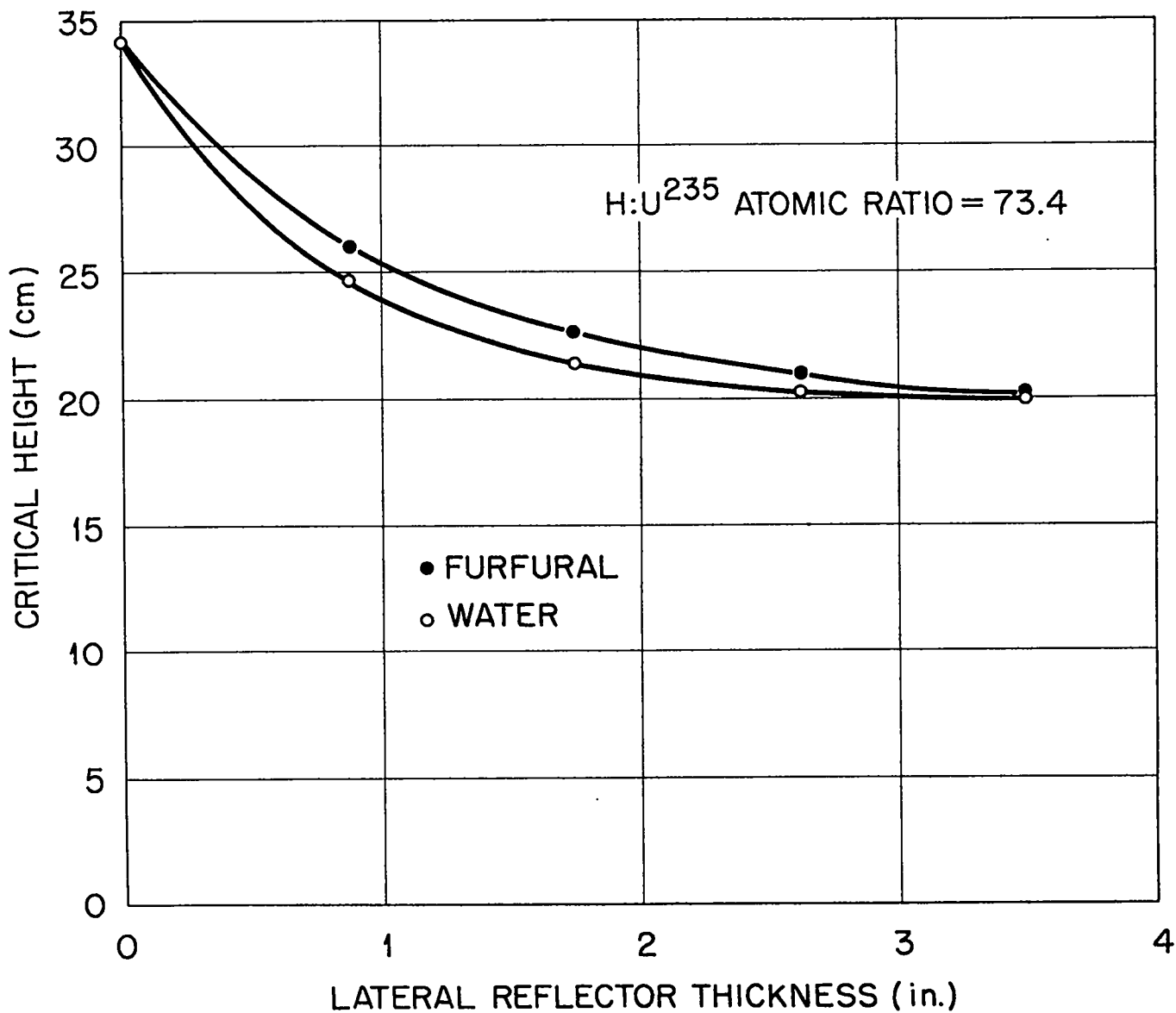
Ratios of critical masses of cylinders (height h, diameter d) to those for spheres appear vs. h/d in Figure 25 for U^{235} solutions (2) and in Figure 26 for Oy(93.5) metal. (6,32) For extrapolation to broad slabs and long cylinders, the following alternative representation is more convenient. The interrelationships between critical cylinders of various height/diameter may be given in terms of effective extrapolation lengths, σ_c , which satisfy

$$\left(\frac{2.405}{\frac{d}{2} + \sigma_c} \right)^2 + \left(\frac{\pi}{h + 2\sigma_c} \right)^2 = B_s^2$$

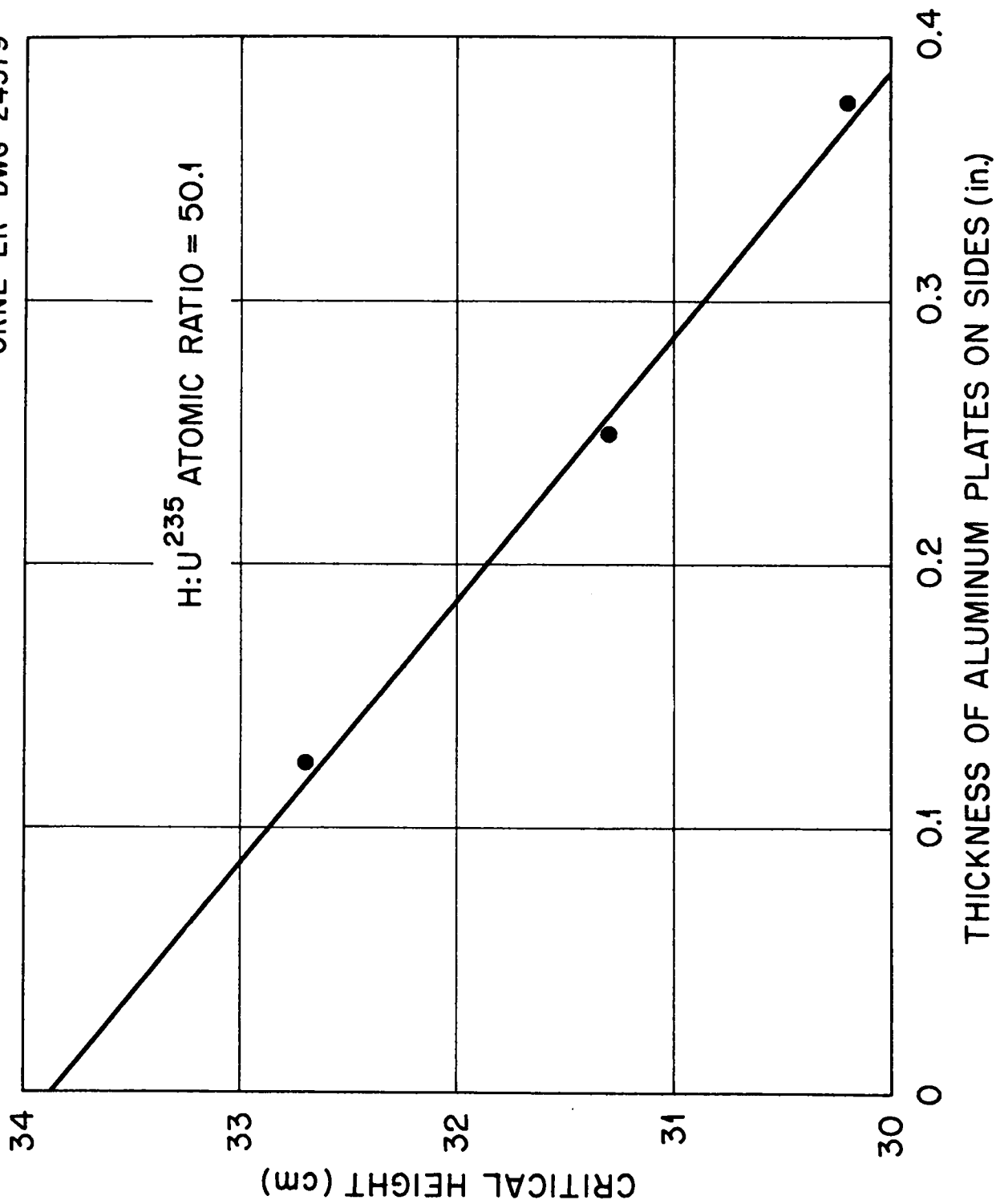
where B_s^2 is an assumed constant buckling (e.g., that of the corresponding sphere). Such extrapolation lengths are shown by Figure 27 for families of solution cylinders that are either bare or water-reflected, and similarly by Figure 28 for metals. (52)

Other shapes

Investigations of the possibility of large-volume solution storage in annular cylinders led to the data of Figures 29 and 30, which apply to critical annuli with inner cylinder Cd-lined and water-filled. (5,53) Similar data exist for solution annuli with internal water but no Cd, and without either water or Cd.



Critical Height as a Function of the Thickness of a Water or Furfural Reflector on the Lateral Surface of a 10-in.-dia Aluminum Cylinder Containing an Enriched U²³⁵ Solution.



CRITICAL HEIGHT of a 6-in.-thick SLAB of U²³⁵ SOLUTION as a FUNCTION of THICKNESS of AL on EACH FACE of the SLAB

CRITICAL VOLUMES OF CYLINDERS OF $C_1H_9O_2$ SOLUTION
RELATIVE TO SPHERICAL VALUES

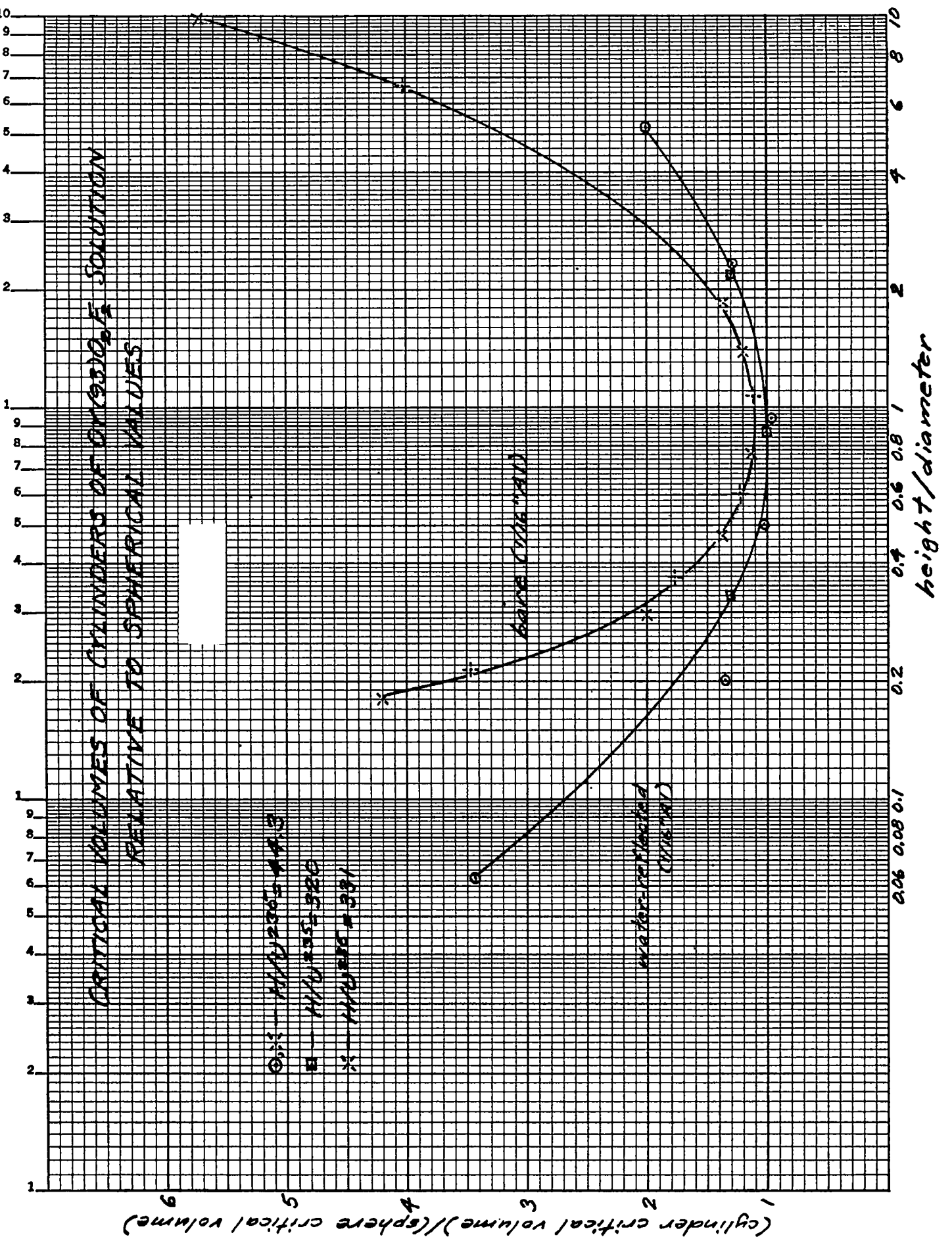


Fig. 25. - 40 -

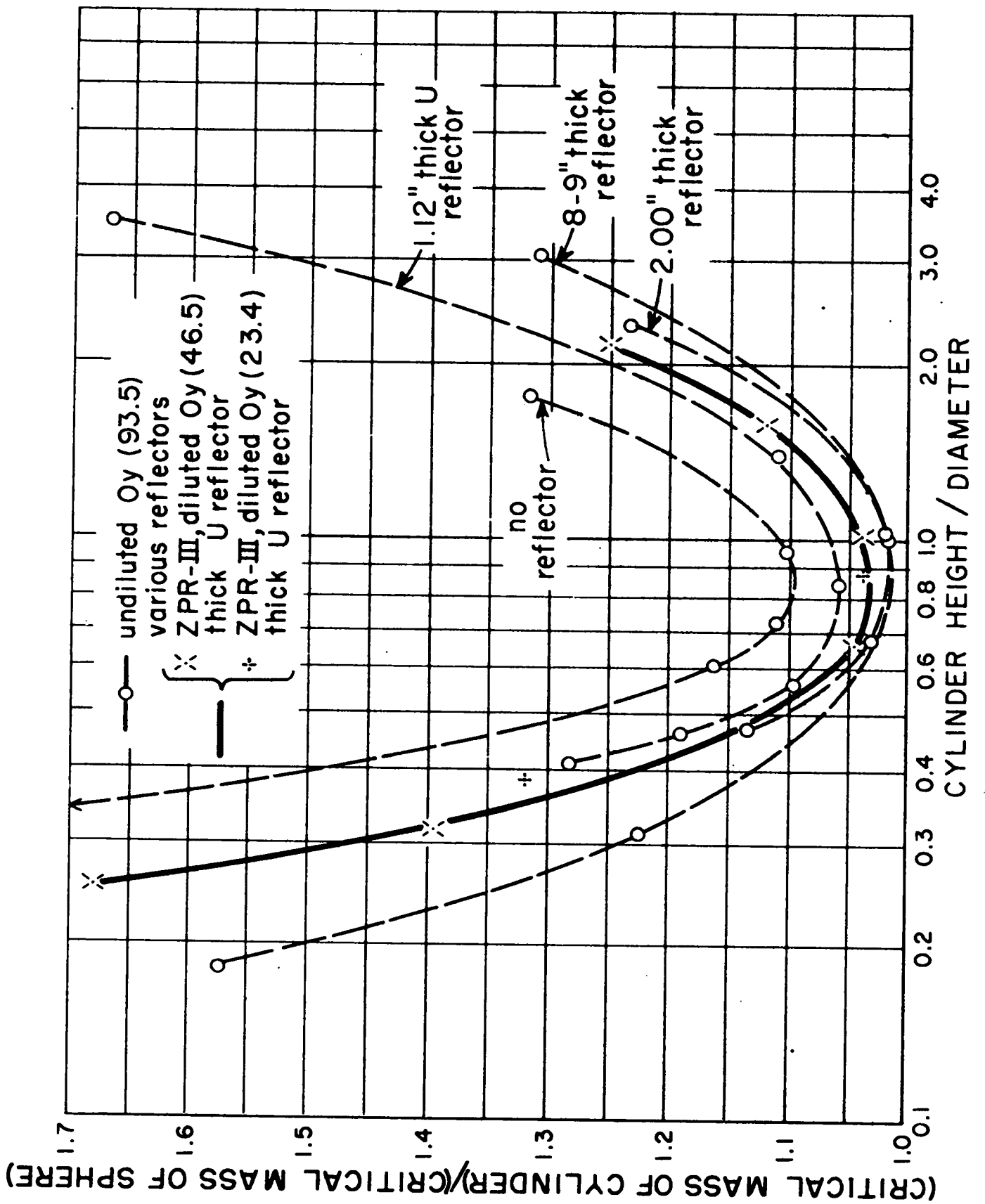


Fig. 26. - 41 -

EFFECTIVE EXTRAPOLATION LENGTHS FOR CYLINDERS OF O_2 (99.2%) F_2 SOLUTION



Fig. 27. - 42 -

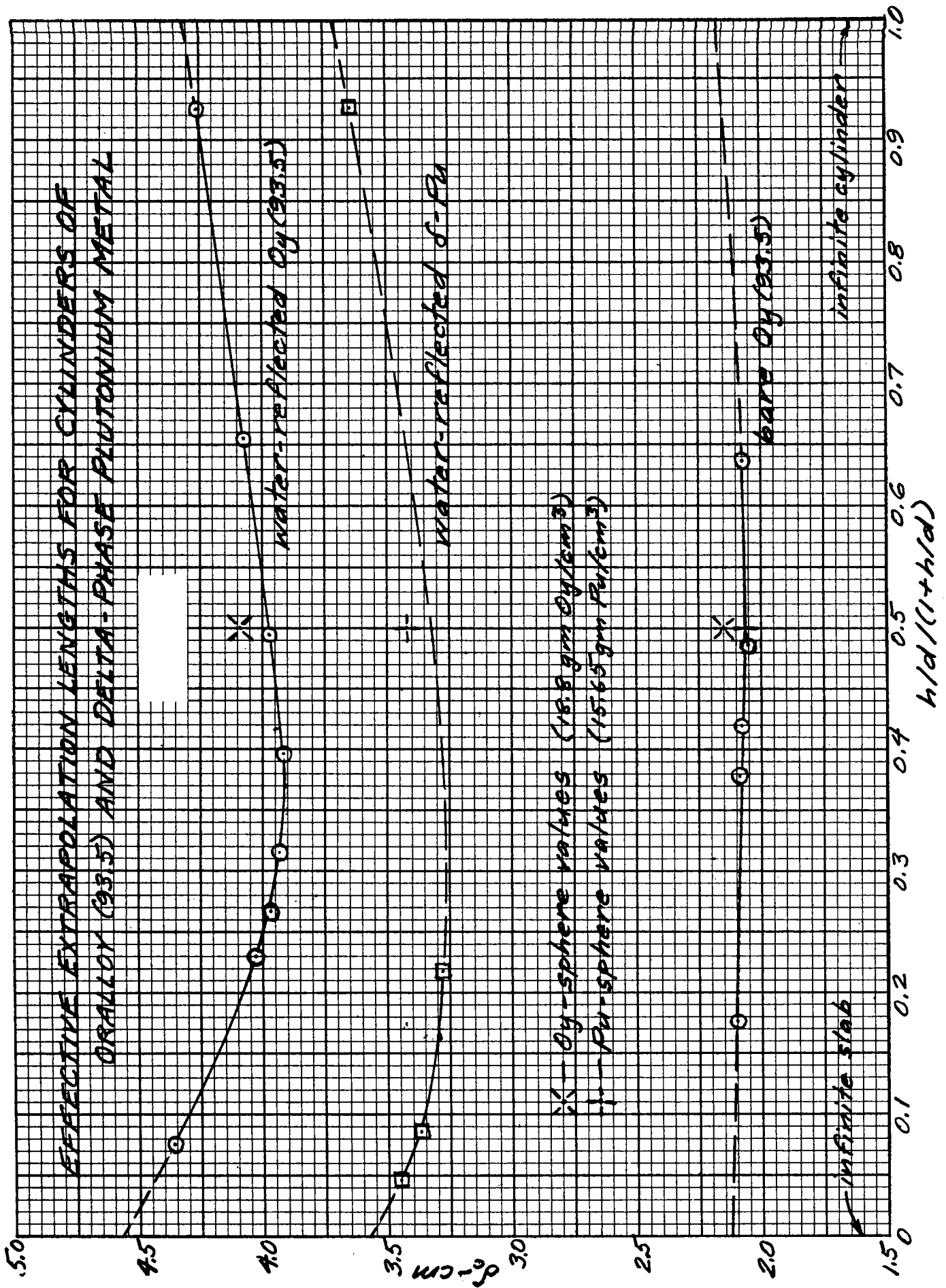
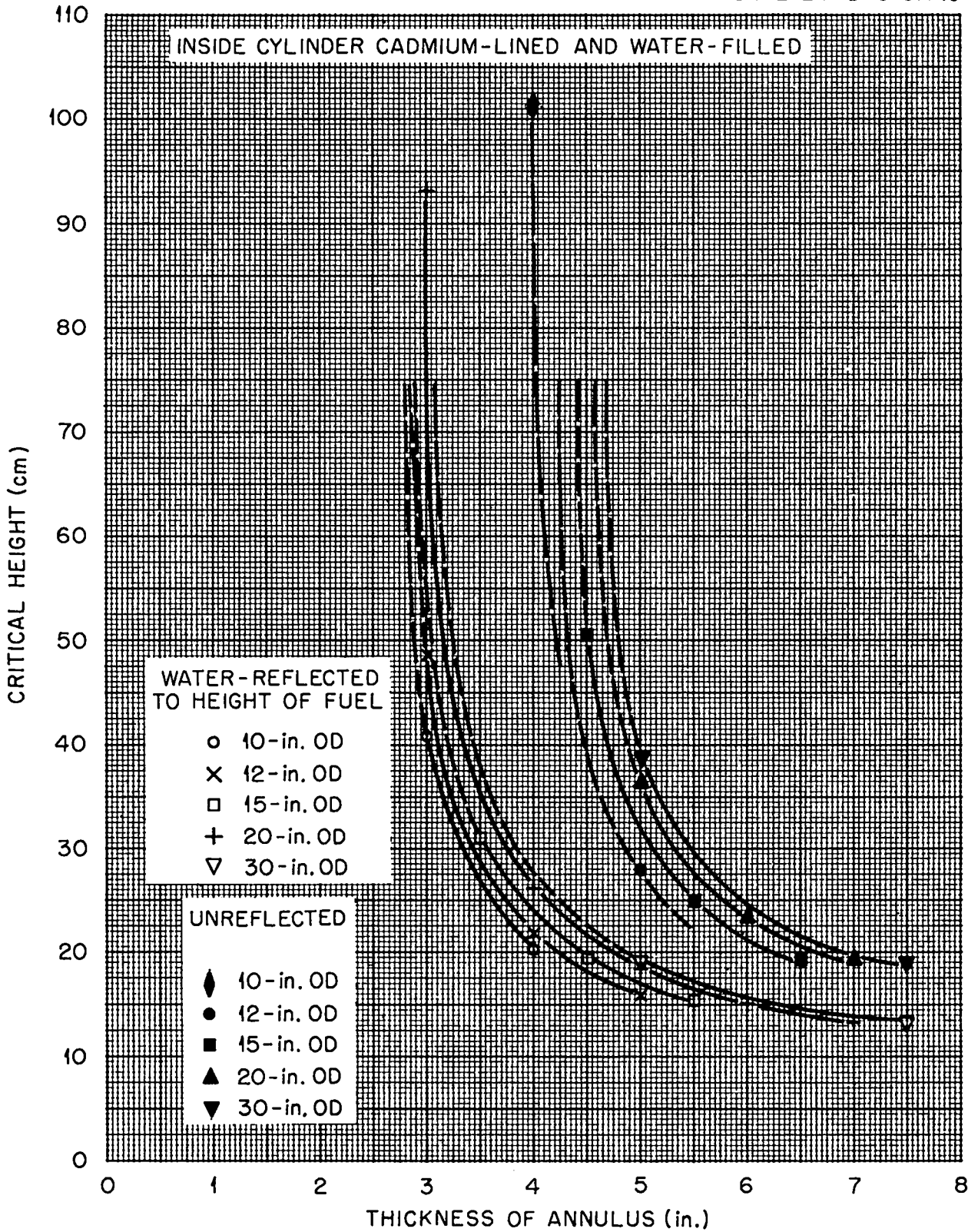
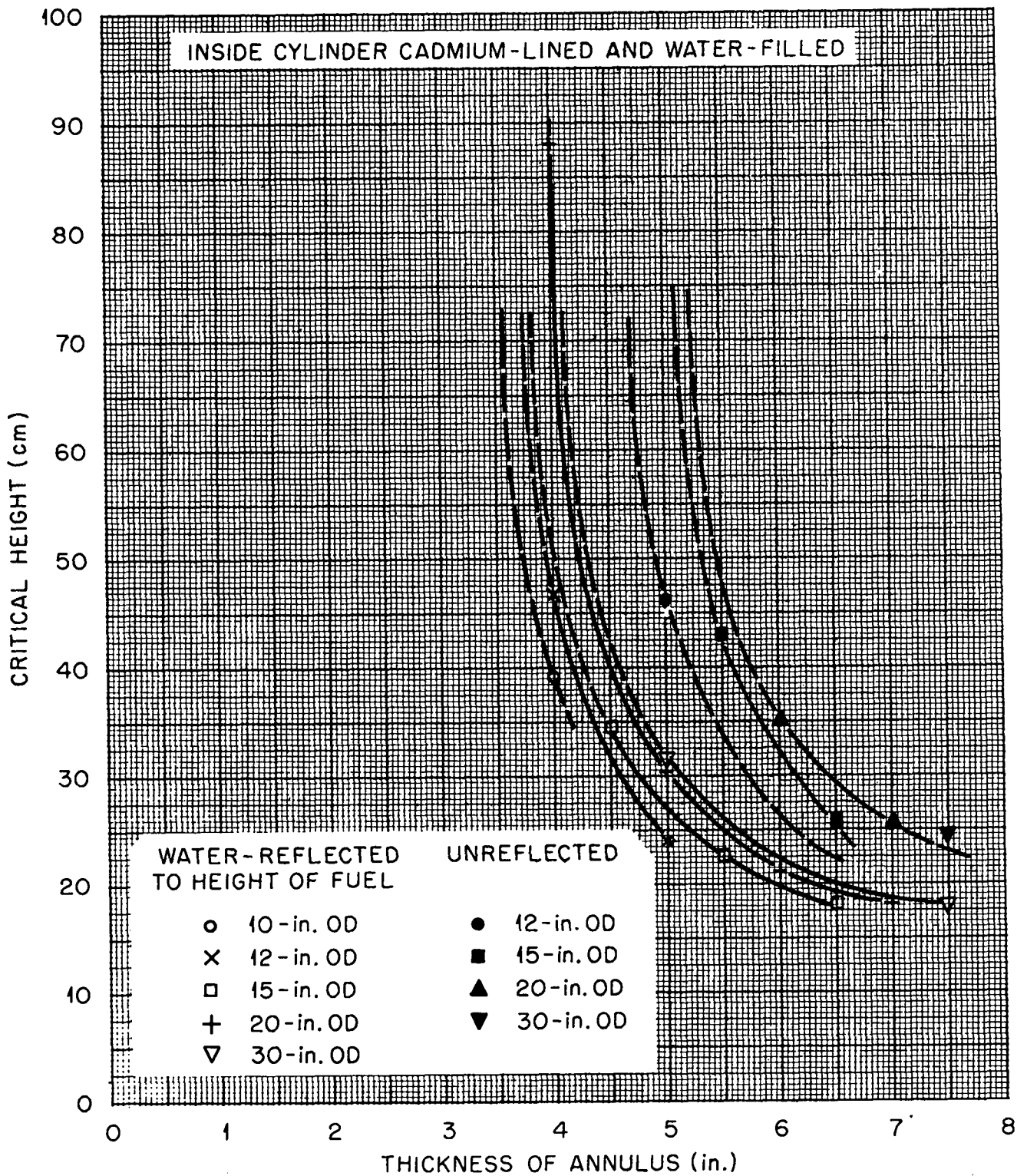


Fig. 28.



Critical Heights of Cylindrical Annuli Containing Aqueous Solutions of 93.2% U^{235} -Enriched Uranyl Fluoride as a Function of the Thicknesses of the Annuli: $H:U^{235}$ Atomic Ratio = 50.4



Critical Heights of Cylindrical Annuli Containing Aqueous Solutions of 93.2% U^{235} -Enriched Uranyl Fluoride as a Function of the Thicknesses of the Annuli: H:U²³⁵ Atomic Ratio = 309.

Table IV gives results of a few observations on critical Oy(~ 93) metal annuli in various reflectors.

Critical and subcritical data on several solution-filled crosses and diagonal pipe intersections appear in Table V. ⁽⁵⁾

Metal-solution systems

Figure 31 shows the relation between critical thickness of a 10" x 16" slab of Oy(~ 90) metal vs. U^{235} concentration of a uranyl nitrate solution in which the slab is immersed. ⁽⁵⁴⁾ The solution is a cylinder 30" diam. x 28" high. From these data and measurements on 16" x 20" slabs, the curves of Figure 32 for slabs in infinitely-thick solution have been deduced. With the 10" x 16" slab, 1 gm Cd per liter of solution (as cadmium nitrate) compensates for 7 gm U^{235} per liter.

The critical thickness of a 5" x 8" slab of Oy(~ 90) metal on the axis of a 9.45" diam. x 16" cylinder of uranyl nitrate solution appears in Figure 33 vs. U^{235} concentration in the solution. ⁽⁵⁵⁾

Some subcritical observations

Numerous multiplication measurements, while not establishing actual critical configurations, have been sufficient to show that certain systems are subcritical. A few conservatively subcritical systems that help fill gaps in critical data follow (others appear in Tables I and V).

1a) Close-packed array of 4 polyethylene containers (7-1/4" ID x 1/4" wall) containing 7"-deep Oy(~ 93) O_2F_2 solution at $H/U^{235} = 260$ (480 gm U^{235} per container), standing on stainless-steel floor of a hood. ⁽⁵⁶⁾

1b) Close-packed array of 6 of the units of 1a) after precipitation of the uranium as a 2-1/4"-deep peroxide layer at $H/U^{235} \sim 75$; 5"-thick uranium-free solution above the peroxide. Apparently less reactive than 1a).

TABLE IV.
 CRITICAL MASSES OF 12-1/4" OD x 6" ID ANNULI
 OF OY (~ 93) METAL

reflector (material, thickness)	critical mass (kg Oy)	critical height (in. Oy)
1" normal U, complete (some excess)	82.7 ± 0.3	3.01
3" normal U, complete	55.9 ± 0.3	2.03
3" polyethylene, complete	60.6 ± 0.3	2.20
2" CS-312 graphite (inner cyl. completely filled)	78.5 ± 0.3	2.86
2" graphite crucible, same as last except without top reflector (wall extends 5" above base of Oy)	97 ± 2	3.54
1" normal U in 2" polyethylene, complete	54.5 ± 0.3	1.98
1" normal U in 2" polyethylene, no reflector in inner 6" cyl.	60.8 ± 0.3	2.21

TABLE V.
 CRITICAL PARAMETERS OF ENRICHED U²³⁵ SOLUTIONS IN
 CYLINDRICAL 60° "Y" AND 90° "CROSS" GEOMETRIES

<u>diameter of cylinders (in.)</u>	<u>geometry</u>	<u>H/U²³⁵ atomic ratio</u>	<u>kg U²³⁵ per liter</u>	<u>critical height (in.)^a</u>
effectively infinite water reflector except at top:				
4	cross	44.3	0.538	b
5	cross	44.3	0.538	5.75
5	cross	73.4	0.337	7.8
5	Y	73.4	0.337	15.6
no reflector:				
5	Y	73.4	0.337	b
5	cross	73.4	0.337	b
7.5	cross	44.3	0.538	b
7.5	cross	72.4	0.342	b

^a Above the intersection of the center lines.

^b Extrapolation of the reciprocal source-neutron multiplication curve from an observation taken at least 36 cm above the intersection of the center lines indicates that this vessel will not be critical at any height.

CRITICAL SLAB THICKNESS

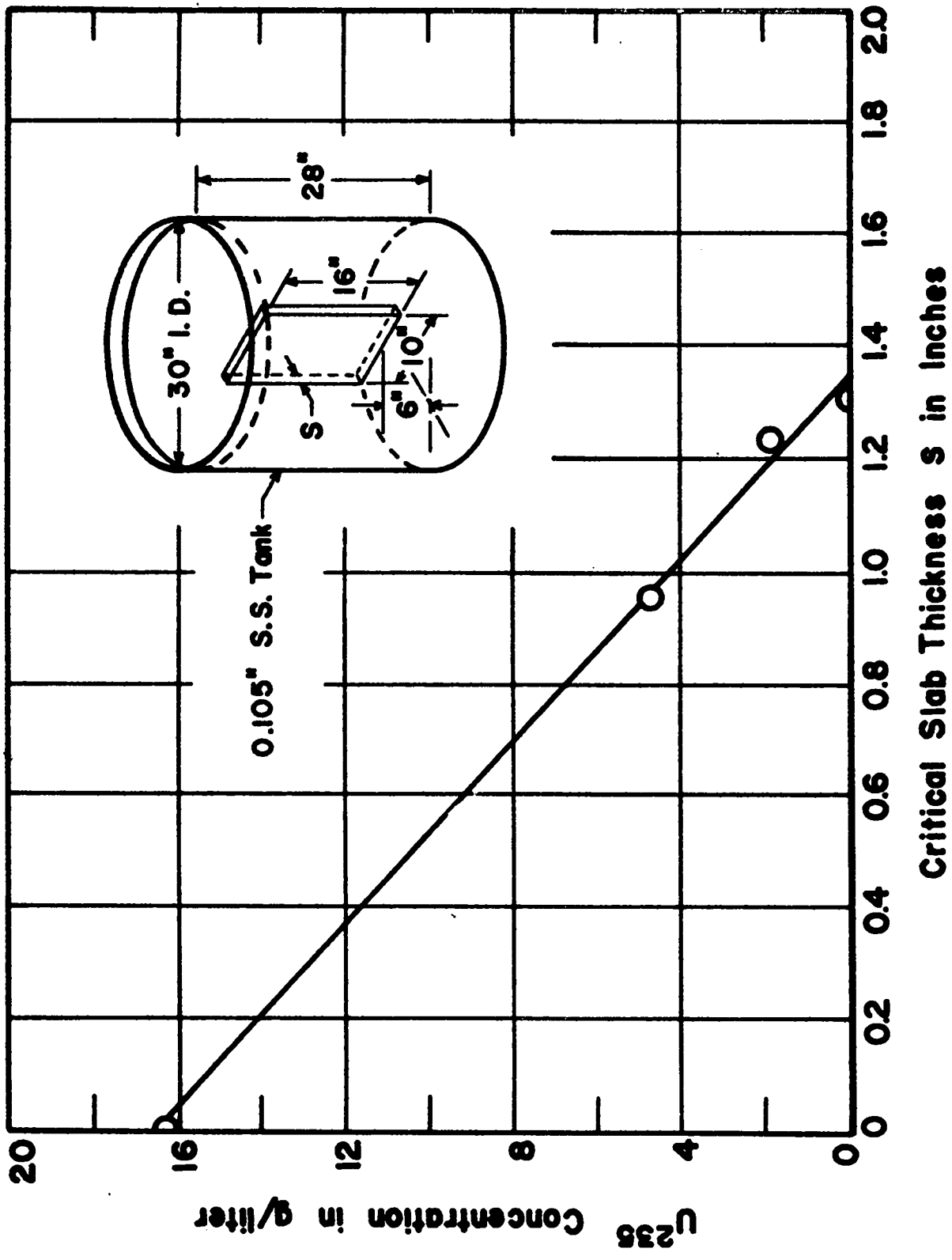


Fig. 31. - 49 -

CRITICAL SLAB THICKNESS IN AN INFINITE SYSTEM

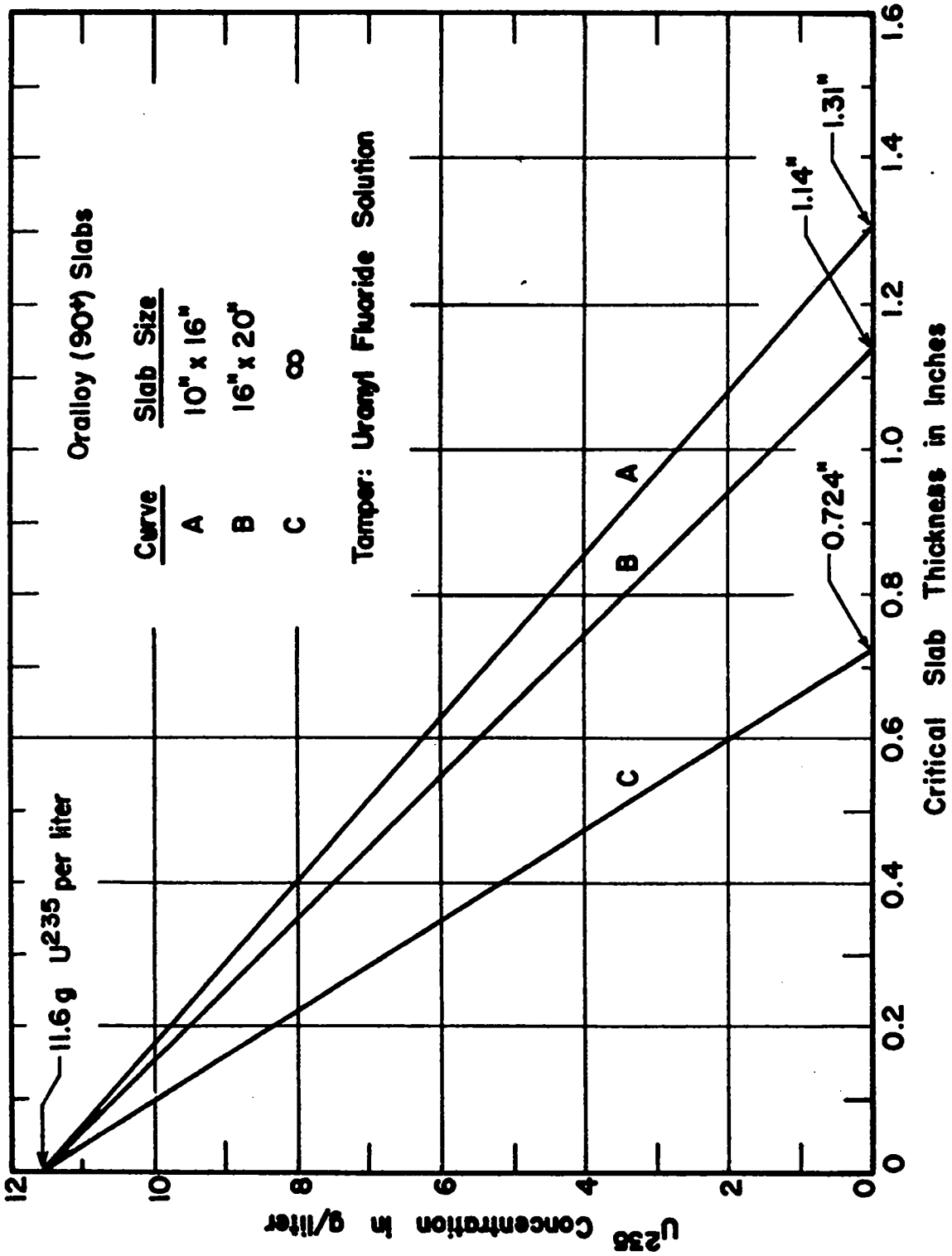


Fig. 32. - 50 -

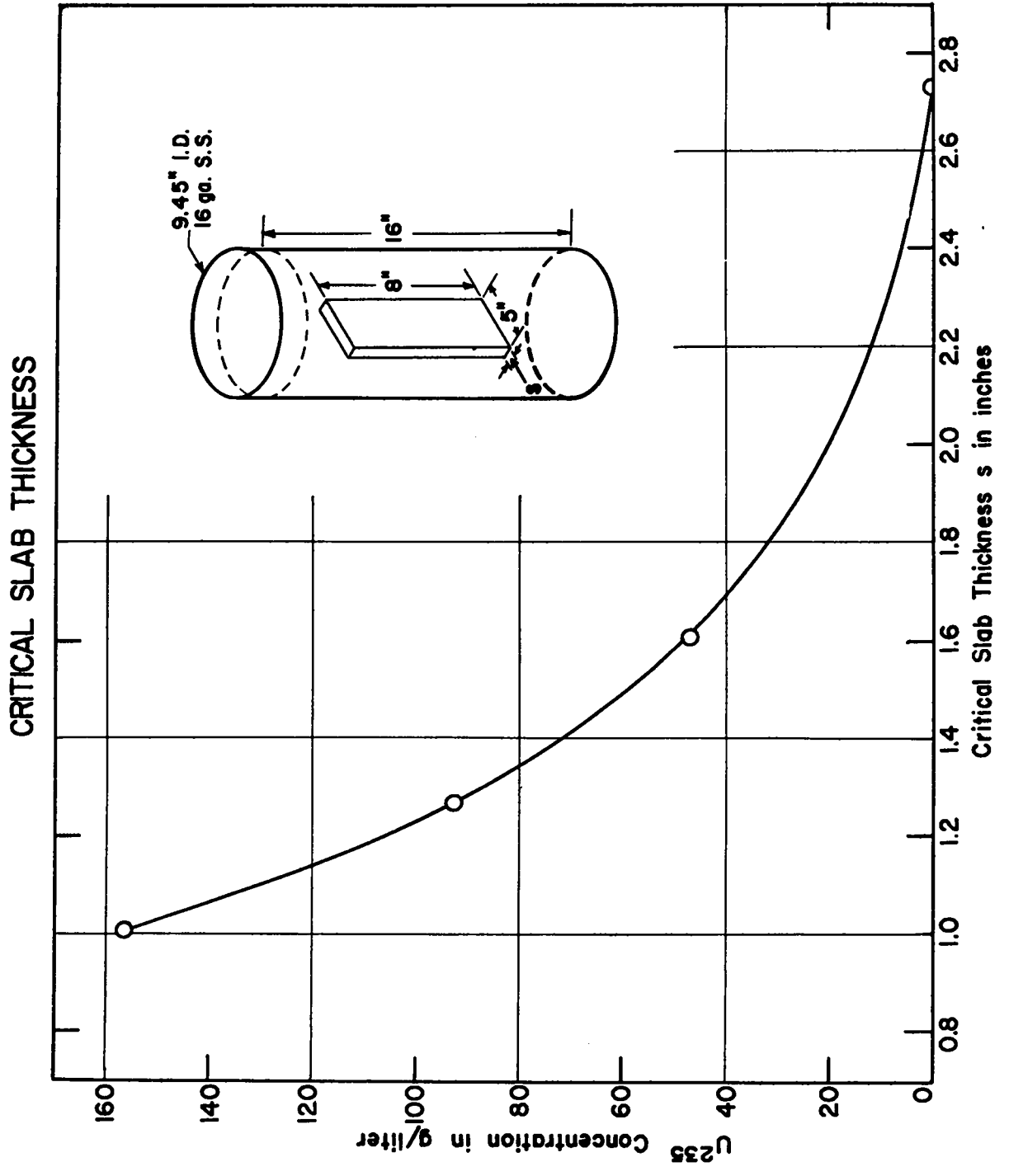


Fig. 33. - 51 -

2) Close-packed array of 17 porcelain filter boats (4" diam.) containing 3-1/2"-deep Oy (~ 93) peroxide at $H/U^{235} \sim 18$; reflected on 3 sides by thick water and concrete. (56)

3) Slab on concrete floor, made up of 23 - 2-3/4" x 2-3/4" x 3-3/4"-deep units of $(Oy \sim 93)_3O_8$ containing water such that $H/U^{235} = 12$ (705 gm U^{235} per unit in milk carton). (56)

4a) Oy (~ 93)-metal slab 8" x 8-1/2" x 1-3/32"-thick, reflected by 6"-thick salt eutectic consisting of 55 w/o K_2CO_3 and 45 w/o Li_2CO_3 . (16" x 17" x 1-3/32" slab also subcritical but at high multiplication). (56)

4b) Stack of four 8" x 8-1/2" x 1-3/2"-thick Oy-metal slabs separated by 2"-thick layers of the salt of 4a), essentially unreflected. Data also exist for Oy (~ 93) sheet distributed in 65 w/o K_2CO_3 , 30 w/o Li_2CO_3 and 5 w/o Na_2CO_3 . (57)

5) Four 30" x 6'-high cylinders of condensed Oy (2%) F_6 at $H/U^{235} \sim 4$, in contact, water reflected. (58)

INTERACTING UNITS

Three-dimensional arrays

Critical data for cubic lattices of fissionable metal units are summarized in Figure 34, where the ordinate is critical capacity of the array in terms of number of bare, spherical critical masses of the material, and the abscissa is volume-fraction F of the lattice that is occupied by the unit. (59) (Consistent densities of units are used for determining coordinates.) Though data do not exist for cubic lattices of nearly equilateral solution units, information about clusters of solution cylinders or slabs can be forced into the form of Figure 34 by confining attention to roughly equilateral lattices ($1/2 \leq h/d \leq 2$). The data of Figure 35 represent this sort

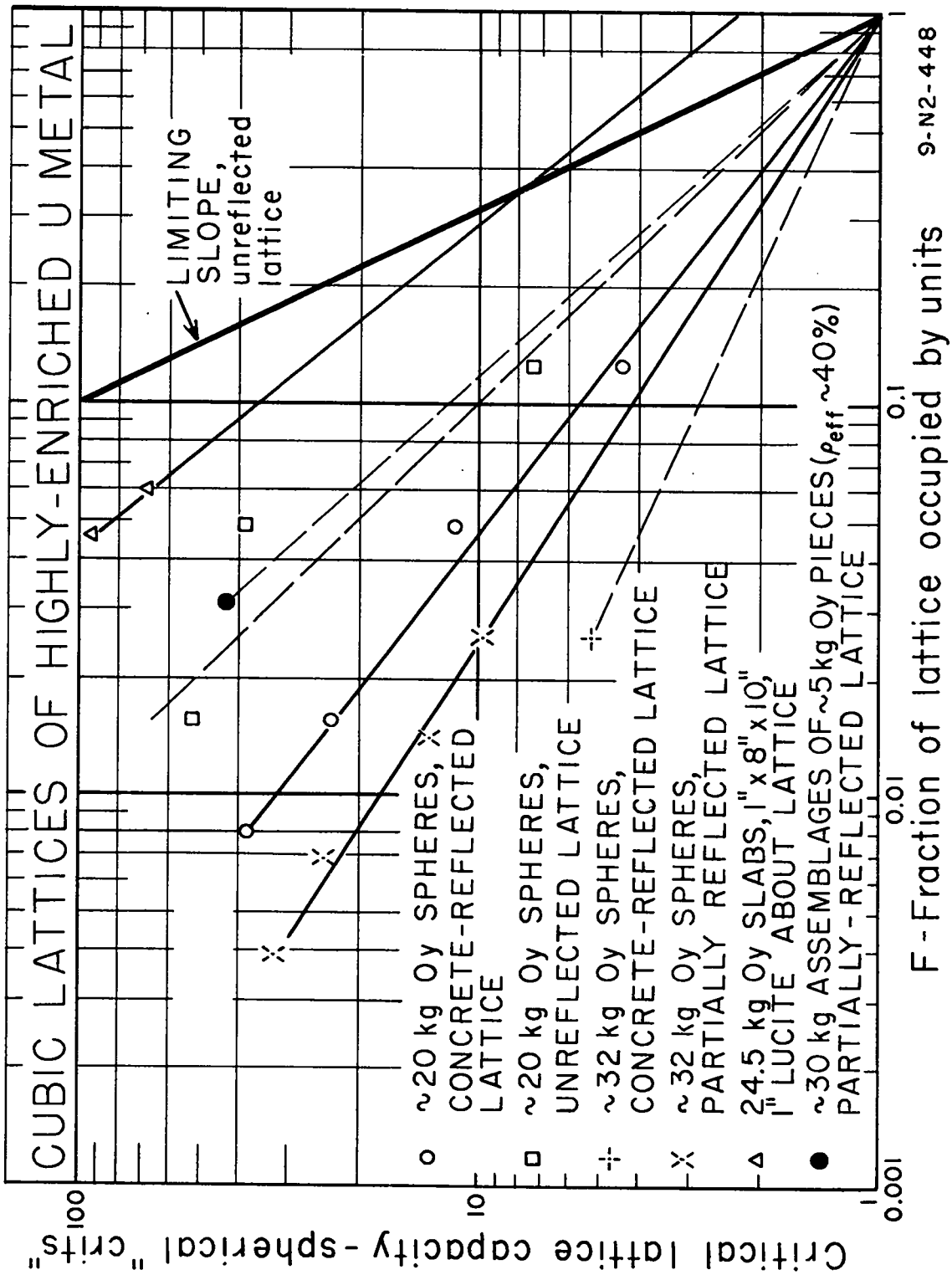
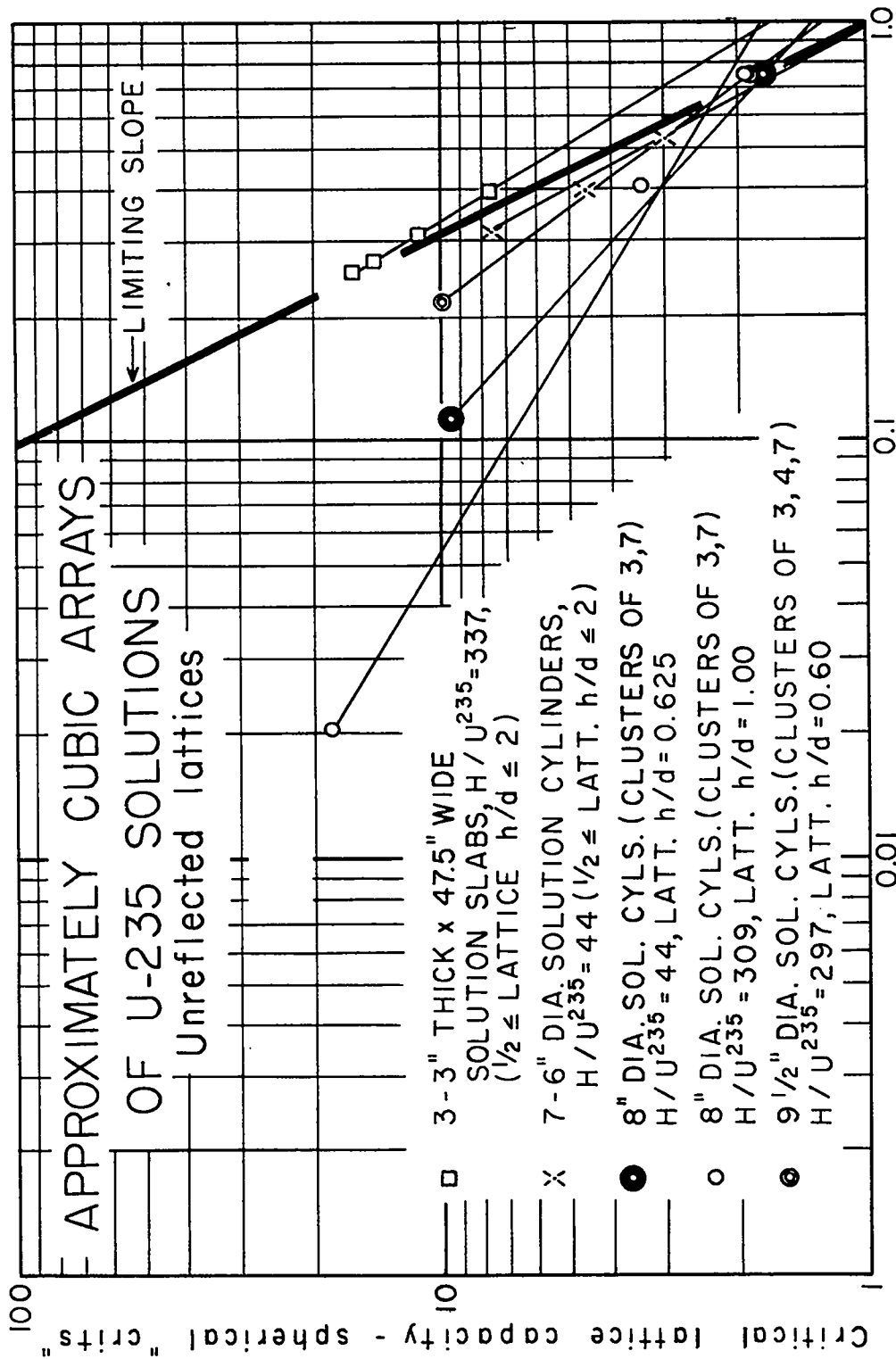


Fig. 34.



R9-N2-446

Fig. 35.

of compromise for 3-3" slabs and 7-6" diam. cylinders. (5,20,21,59)
In the cases of 8"-diam. and 9-1/2" diam. cylinders, where data exist for clusters of different numbers, shape is preserved (assuming lattice extrapolation lengths equal to one-half of the surface-to-surface separation of units).

Each slope, $-s$, of Figures 34 and 35 corresponds to a density exponent if the lattice is thought of as a single low-density unit. Figure 36 is a correlation of s with quantity of reflector about the lattice and reactivity (fraction critical) of an individual unit.

It has been observed that 1"-thick plexiglas between all pairs of 1" x 8" x 10" Oy (~ 93)-metal units decreases the critical number in a cubic lattice by the factor ~ 5 . (60)

Linear and planar arrays

Figure 37 gives cross-multiplication data for linear and two-dimensional arrays of Oy (~ 93)-metal units. It suggests that interactions for large linear or planar arrays can be predicted from measurements on a few units, provided $1-1/M_x$ is an undistorted measure of reactivity. (59)

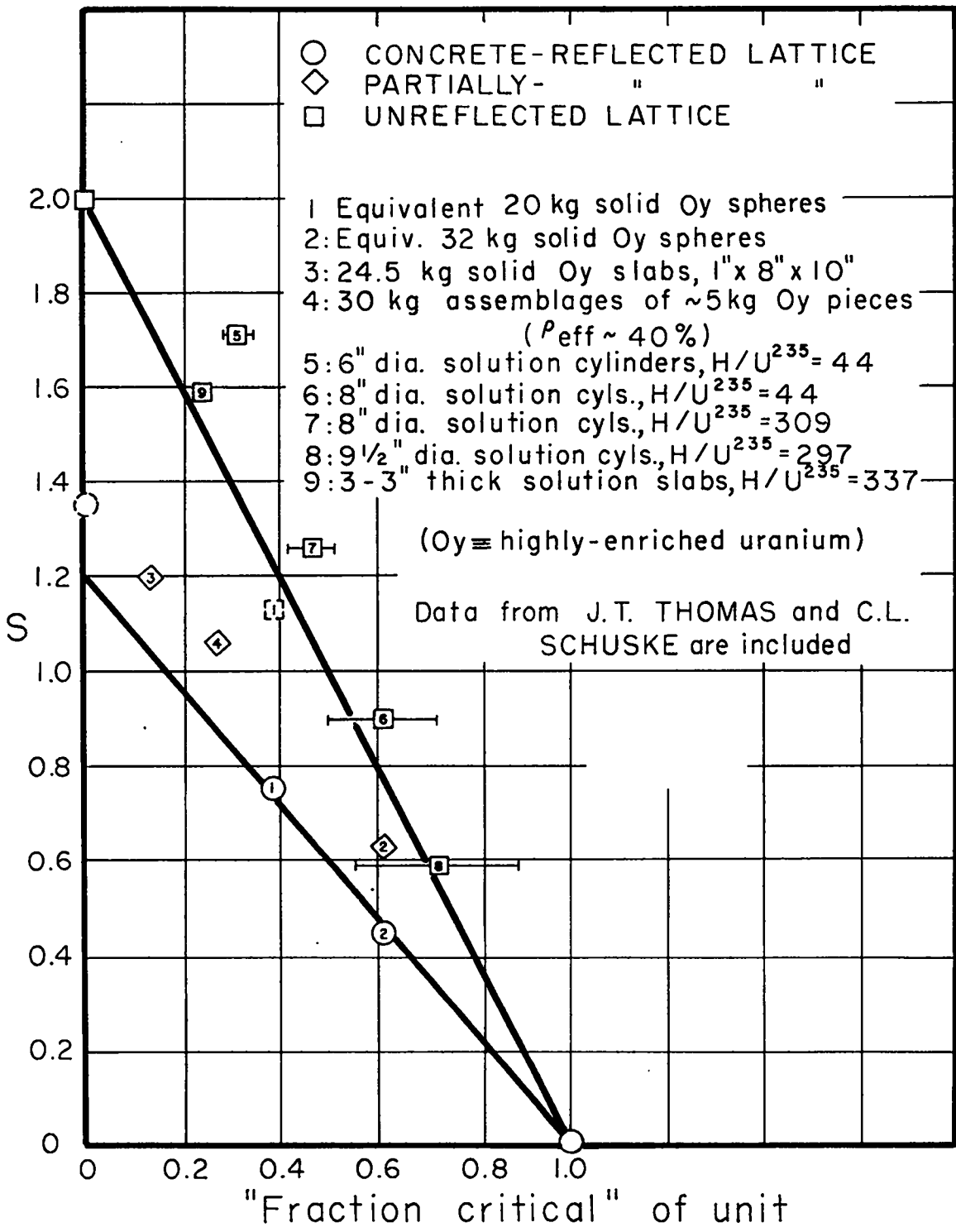
The influence of spacing on interaction between various numbers of bare in-plane solution cylinders is shown in Figure 38.

Pairs of water-immersed units

Figure 39 gives a measure of interaction between pairs of units immersed in water vs. separation of units. Whereas 4"-thick water effectively isolates small spheres, about 8" is required for long cylinders and large slabs that are face to face. (5,6,20,21,61)

Effects of incidental reflectors

Figure 40 shows the critical height of a 9"-diameter U^{235} solution as a function of distance from a concrete slab. (5) Effects of carbon



R9-N2-435

Fig. 36. - 56 -

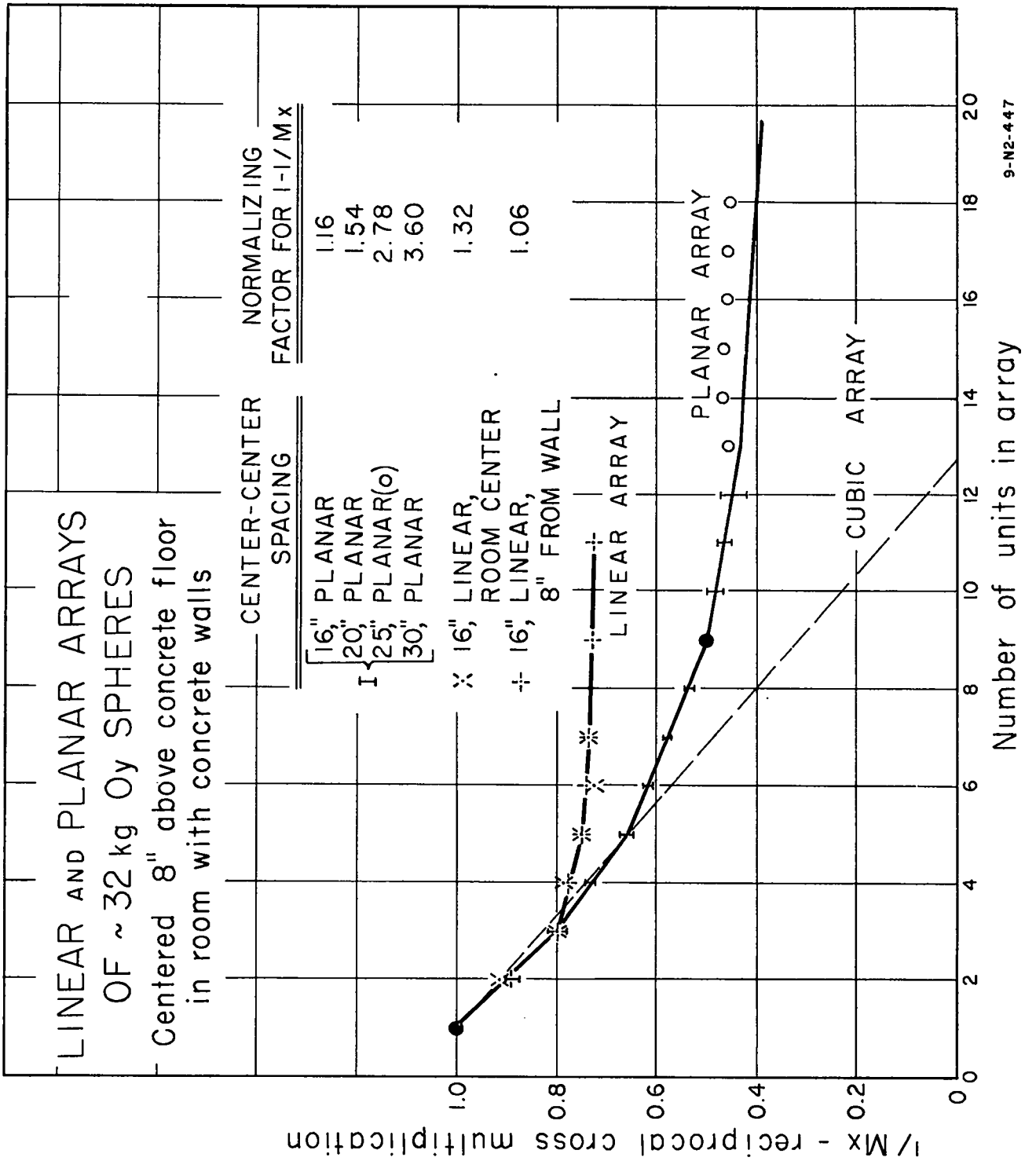


Fig. 37. - 57 -

INTERACTION WITHIN IN-PLANE ARRAY OF
 UNREFLECTED 8" DIAMETER O_2 F_2
 SOLUTION CYLINDERS $H/U^{2.35} = 44.9$

(interaction = $1 - V_e/V_{cs}$, where V_e = equivalent-sphere
 volume of one cylinder in the critical array,
 V_{cs} = spherical critical volume)

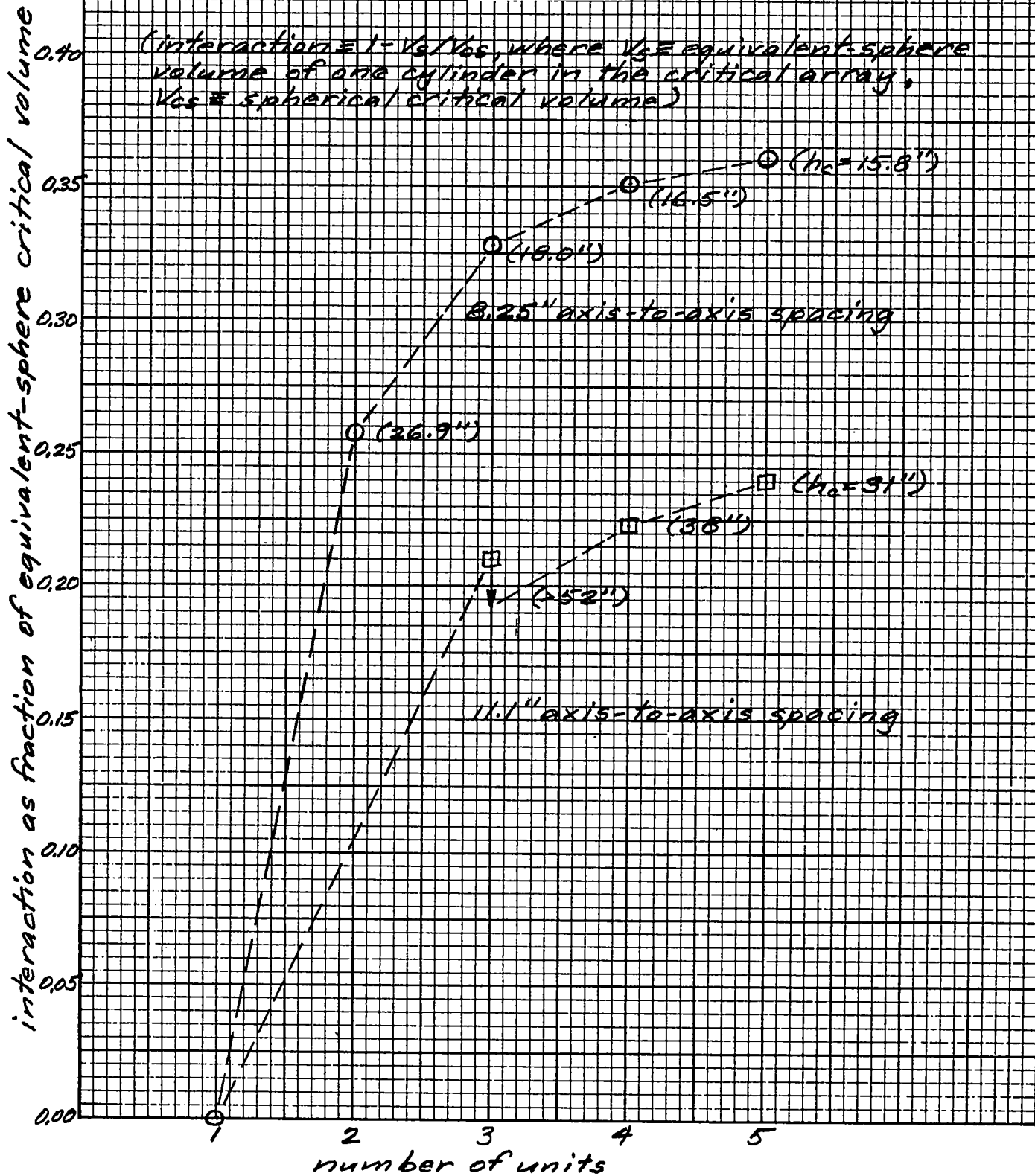


Fig. 38. - 58 -

INTERACTION BETWEEN WATER-FLOODED PAIRS
OF FISSIONABLE UNITS (MAXIMUM AREAS FACE)

- ⊖ ~ 20 kg U_2^{235} metal spheres
- ⊕ 6" diam U_2^{235} solution cylinder, $H/U^{235} = 44.3$
- ⊠ 3" thick x 48" wide U_2^{235} solution slab, $H/U^{235} = 58.1$

(Interaction = $1 - V_0/V_0c$, where V_0 is equivalent sphere volume of a unit, V_0c is corresponding spherical critical volume)

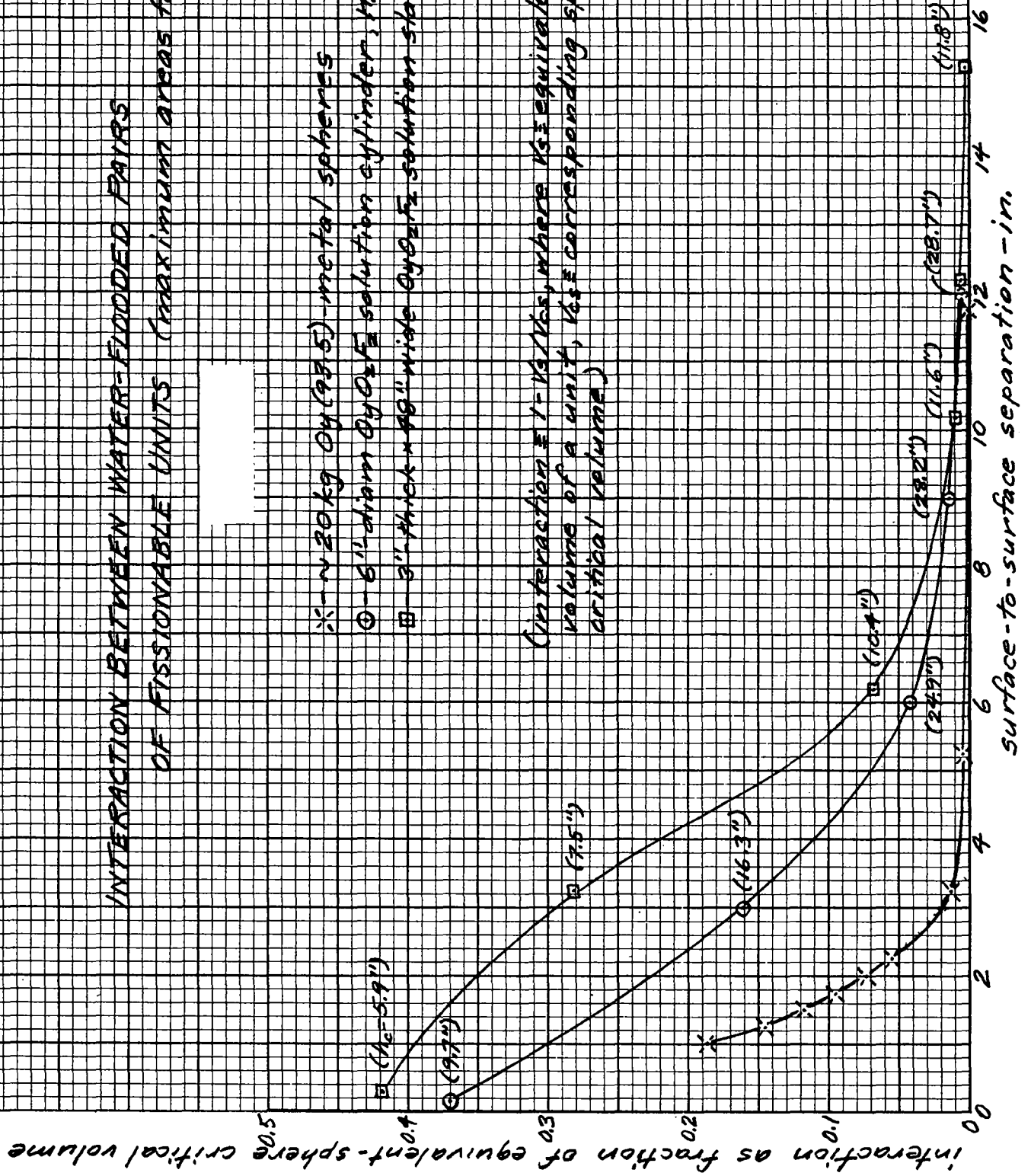
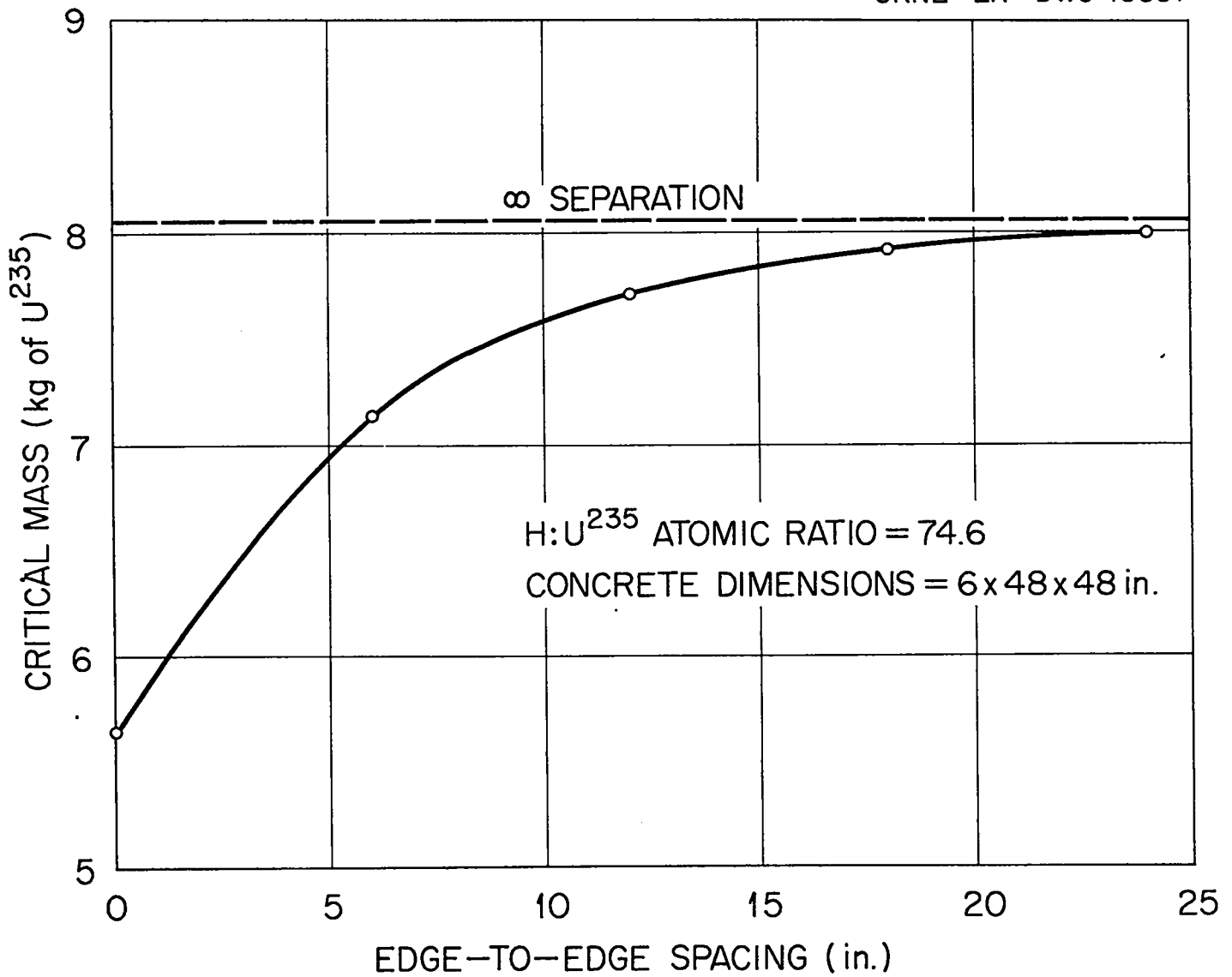


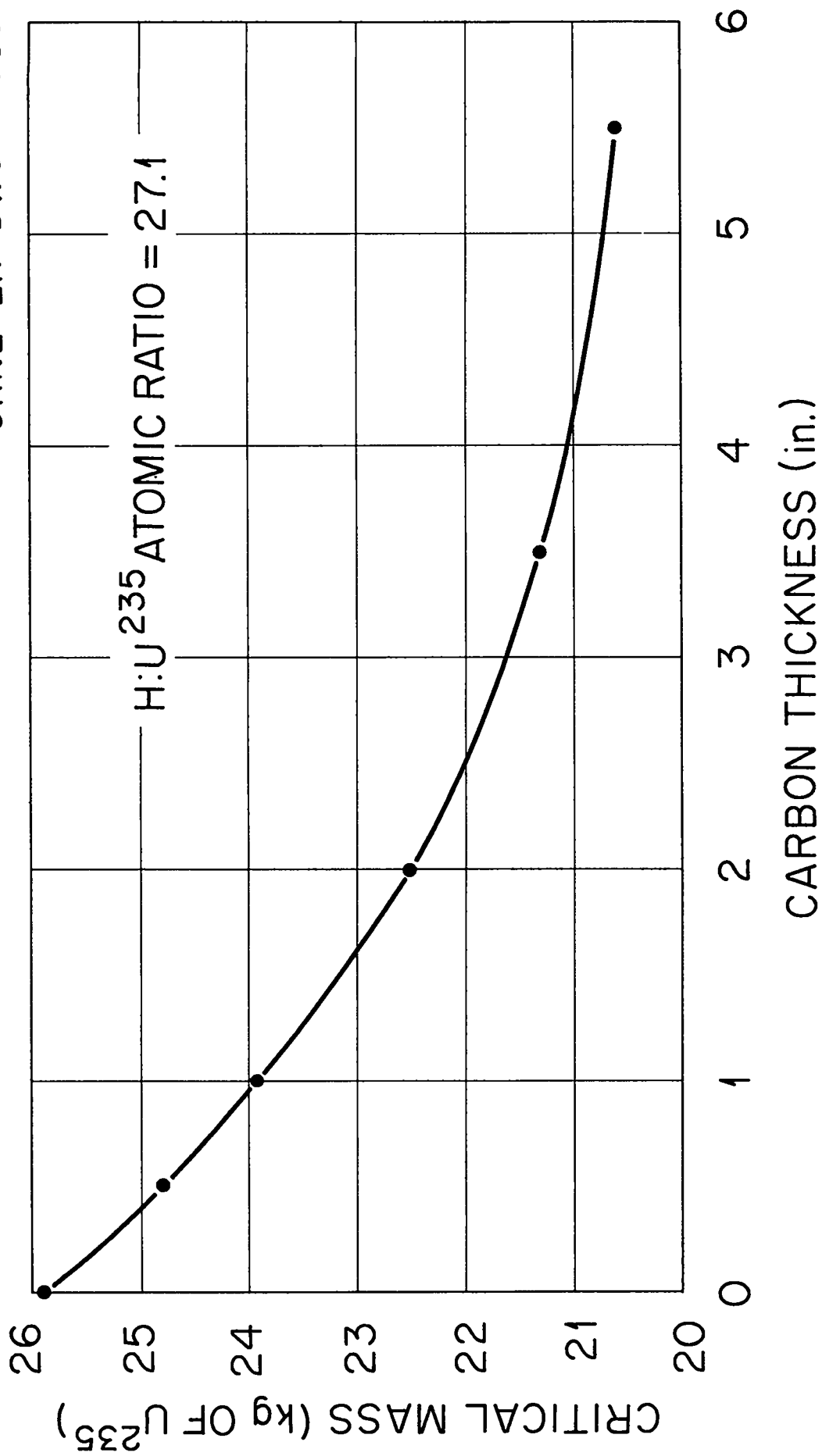
Fig. 39. - 59 -



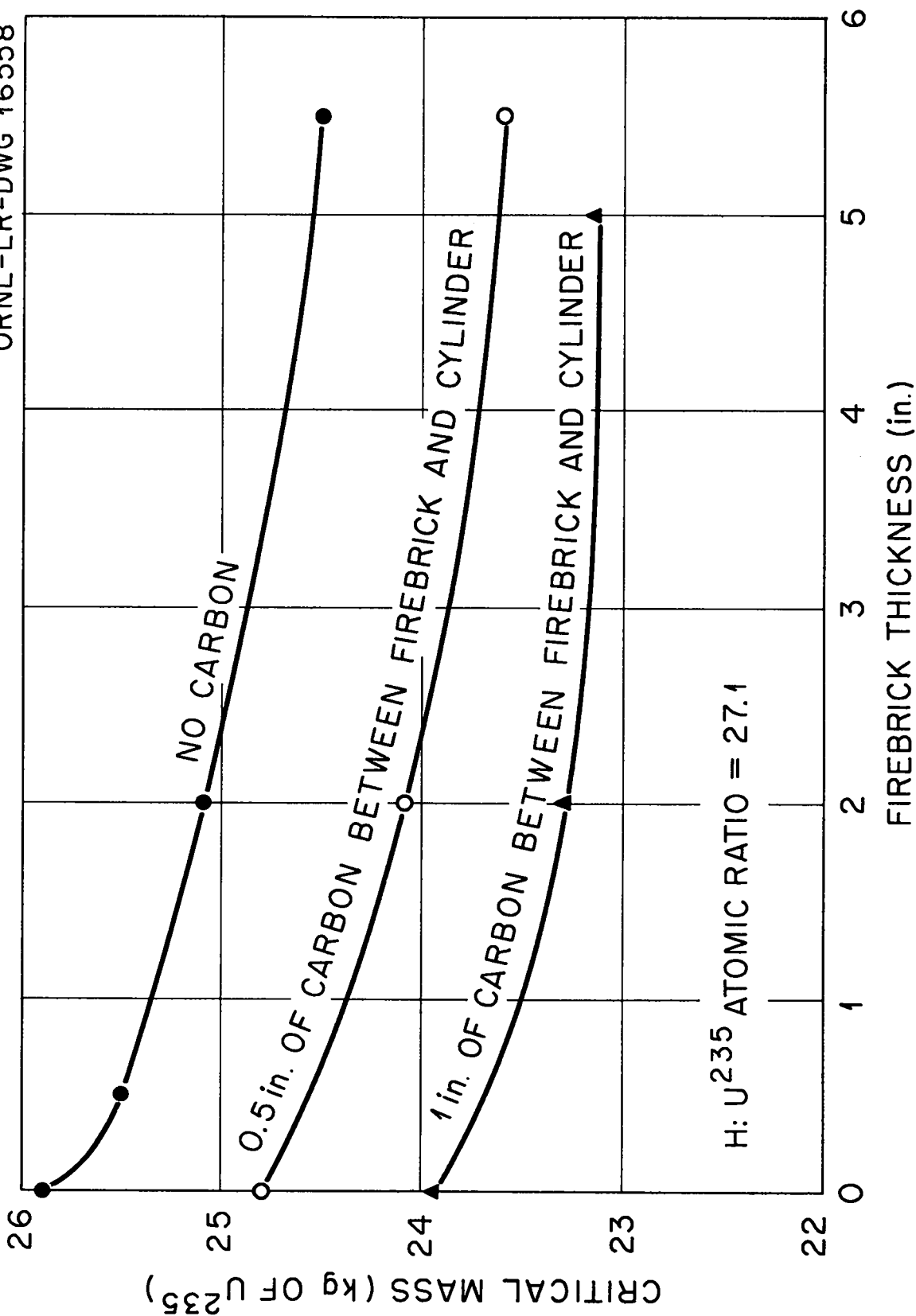
Critical Mass as a Function of the Distance Between a 9-in.-dia Stainless Steel Cylinder Containing an Enriched U^{235} Solution and a 6-in.-thick Concrete Wall.

and firebrick as reflectors on the base of a 20"-diameter U^{235} solution cylinder are given by Figures 41 and 42.

The influence of a concrete wall about 8-1/2" from a vertical plane array of Oy-metal units appears in Figure 43 as a function of concrete thickness. ⁽²²⁾ Figure 44 shows the degree to which a concrete wall of various thicknesses isolates plane arrays of the Oy units 8-1/2" from each side of the wall. The ordinate is ratio of multiplication of the two arrays, with wall between, to that of a single array.



Critical Mass as a Function of the Thickness of Carbon on the Bottom of a 20-in.-dia Aluminum Cylinder Containing an Enriched U²³⁵ Solution.



H: U²³⁵ ATOMIC RATIO = 27.1

Fig. 42.

Critical Mass as a Function of the Thickness of Carbon and Firebrick on the Bottom of a 20-in.-dia Aluminum Cylinder Containing an Enriched U²³⁵ Solution.

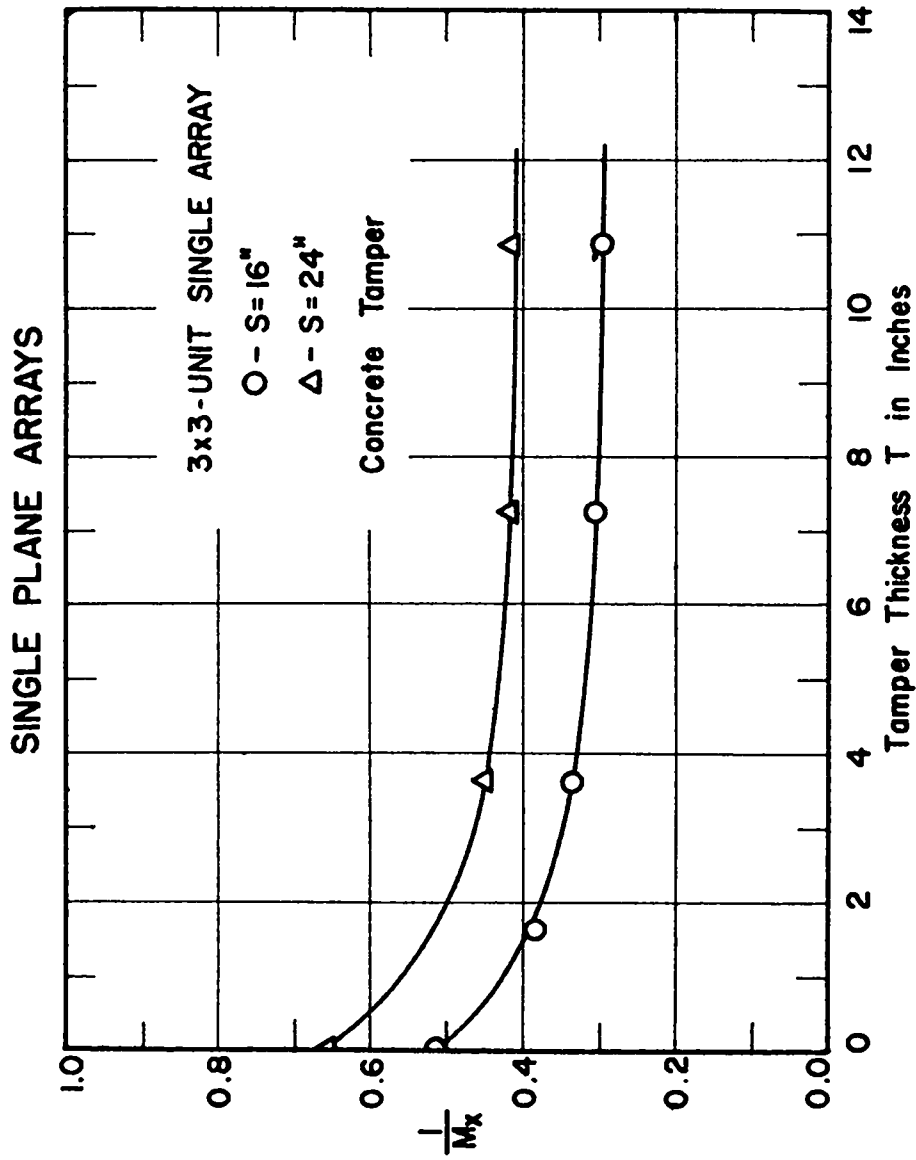


Fig. 43. - 64 -

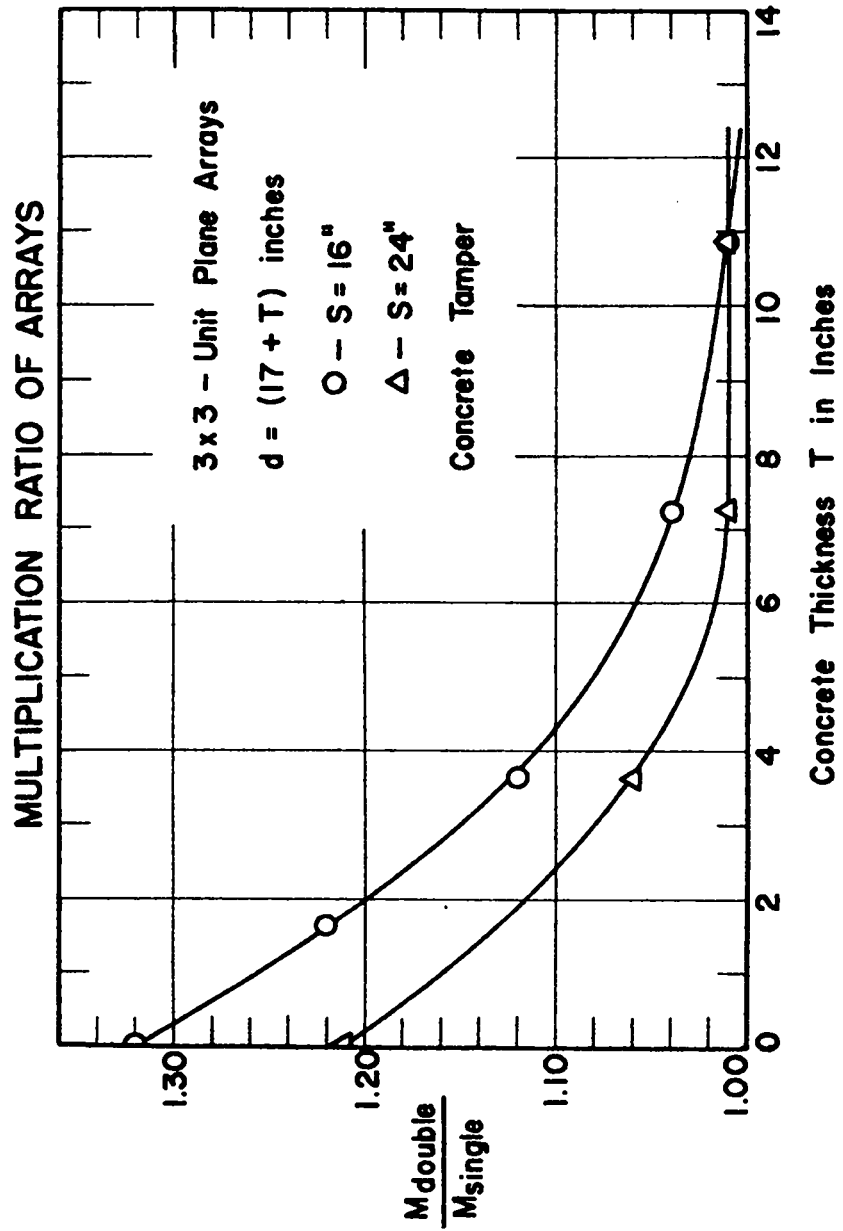


Fig. 44.

REFERENCES

References 1-30 appear in TID-7016, Nuclear Safety Guide.

31. D. W. Magnuson, R. Gwin, Comparison of Critical Experiments for the Determination of Eta of U²³³, Trans. of the Am. Nuclear Soc. 1, No. 1, p. 146 (June 1959).
32. G. E. Hansen, H. C. Paxton, D. P. Wood, Critical Plutonium and Orallo Metal Cylinders of Extreme Shape, to appear in Nuclear Sci. and Eng.
33. Communication from John McEnhill, Aldermaston, England.
34. E. A. Plassmann, D. P. Wood, Critical Reflector Thicknesses for Spherical U²³³ and Pu²³⁹ Systems, to appear in Nuclear Sci. and Eng.
35. J. C. Hoogterp, Critical Masses of Orallo Lattices Immersed in Water, LA-2026, March 1957.
36. R. E. Heineman, Experience in the Use of the Physical Constants Testing Reactor, Paper UN-1929, Second International Conf. on the Peaceful Uses of Atomic Energy, September 1958.
37. J. J. Lynn, J. T. Mihalcz, W. C. Connelly, Homogeneous Hydrogen Moderated Critical Assemblies with Two Percent U²³⁵ Enriched Uranium, ORNL-2609, 1958.
38. D. Callihan, D. F. Cronin, ORNL-CF-55-10-97, October 1955 (Classified).
39. H. Kouts, Lattices of Slightly Enriched Uranium in Ordinary Water, Paper UN-1841, Second International Conf. on the Peaceful Uses of Atomic Energy, September 1958.
40. D. Callihan, et al, Physics Division Semiannual Progress Report for Period Ending March 10, 1954, ORNL-1715, July 1954 (Classified).
41. L. W. Gilley, A. D. Callihan, Nuclear Safety Tests on a Proposed Ball Mill, ORNL-54-9-89, September 1954.
42. J. K. Fox, L. W. Gilley, Critical Parameters for 20-in.-dia. Stainless Steel Cylinders Containing Aqueous Solutions of U²³⁵ Poisoned with Pyrex Glass, Neutron Physics Division Annual Progress Report for Period Ending September 1, 1959, ORNL-2842, pp. 78-81.

43. F. H. Langell, et al, Rocky Flats Technical Quarterly Progress Report for January, February, March 1959, RFP-151, p. 44 (Classified).
44. L. B. Engle, G. E. Hansen, H. C. Paxton, Reactivity Contributions of Various Materials in Topsy, Godiva and Jezebel, to appear in Nuclear Sci. and Eng.
45. R. N. Olcott, Homogeneous Heavy Water Moderated Critical Assemblies. Part I. Experimental, Nuclear Sci. and Eng. 1, No. 4, p. 327, August 1956.
46. J. E. Schwager, F. A. Kloverstrom, W. S. Gilbert, Critical Measurements on Intermediate-Energy Graphite-U²³⁵ Systems, UCRL-5006, November 1957.
47. F. A. Kloverstrom, R.M.R. Deck, A. J. Reyenga, Critical Measurements on Near-Homogeneous BeO-Moderated, Oralloid-Fueled Systems, UCRL-5369 Pt. 1, July 1959.
48. D. P. Wood, C. C. Byers, L. C. Osborn, Critical Masses of Cylinders of Plutonium Diluted with Other Metals, to appear in Nuclear Sci. and Eng.
49. F. A. Kloverstrom, Spherical and Cylindrical Plutonium Critical Masses, UCRL-4957, September 1957.
50. H. R. Ralston, UCRL-4457, February 1955 (Classified).
51. H. R. Ralston, UCRL-5610, June 1959 (Classified).
52. G. E. Hansen, Properties of Elementary Fast-Neutron Critical Assemblies, Paper UN-562, International Conference on the Peaceful Uses of Atomic Energy, September 1958.
53. J. K. Fox, L. W. Gilley, Critical Parameters for Poisoned Annular Cylinders Containing Aqueous Solutions of U²³⁵, ORNL-CF-58-8-5, August 1958.
54. C. L. Schuske, M. G. Arthur, D. F. Smith, RFP-66, August 1956 (Classified).
55. C. L. Schuske, M. G. Arthur, D. F. Smith, RFP-69, October 1956 (Classified).
56. C. L. Schuske, M. G. Arthur, D. F. Smith, RFP-58, January 1956 (Classified).

57. C. L. Schuske, M. G. Arthur, A. Goodwin, Jr., A. N. Nickel, D. F. Smith, RFP-89, November 1957 (Classified).
58. A. D. Callihan, A Test of Neutron Multiplication by Slightly Enriched Uranium, Part II, ORNL-1698, March 1954.
59. H. C. Paxton, Standards for Fissionable Materials Outside Reactors, STANDARDS WORK IN THE AMERICAN NUCLEAR SOCIETY, A Special Session of Invited Papers, pp. 55-63, June 1959.
60. J. T. Mihalcz, J. J. Lynn, Multiplication Measurements with Slabs of Enriched Uranium, Neutron Physics Division Annual Progress Report for Period Ending September 1, 1959, ORNL-2842, pp. 67-68.
61. C. L. Schuske, A. N. Nickel, Isolation Thickness of Water for UO_2F_2 Solution Systems, RFP-169, October 1959.